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신고리 3,4호기 최종안전성분석보고서

표 1.8-2 (20 중 1)

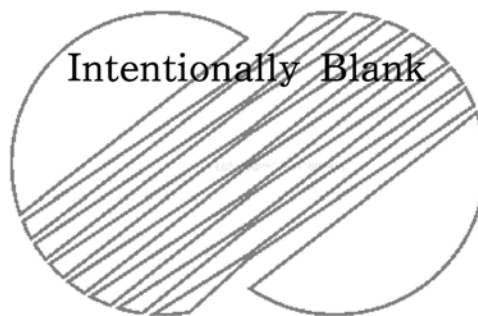
신고리 3,4호기에 대한 NRC Generic Letter의 적용성 분석

번 호	제 목	비 고
08-1	Managing Gas Accumulation in Emergency Core Cooling, Decay Heat Removal, and Containment Spray Systems	6
07-1	Inaccessible or Underground Power Cable Failures That Disable Accident Mitigation Systems or Cause Plant Transients	8.3
06-03	Potentially Nonconforming Hemyc and MT Fire Barrier Configurations	9.5.1
06-02	Grid Reliability and the Impact on Plant Risk and the Operability of Offsite Power	N/A <sup>2)</sup>
06-01	Steam Generator Tube Integrity and Associated Technical Specifications	N/A <sup>2)</sup>
04-02	Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors	6.8
04-01	Requirements for Steam Generator Tube Inspections	N/A <sup>2)</sup>
03-01	Control Room Habitability	6.4



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신고리 3,4호기 최종안전성분석보고서

표 1.8-2 (20 중 2)

번 호	제 목	비 고
98-05	BWR Licensee Use of the BWRVIP-05 Report to Request Relief from Augmented Examination Requirements on Reactor Pressure Vessel Circumferential Shell Welds	N/A <sup>1)</sup>
98-04	Potential for Degradation of Emergency Core Cooling System and the Containment Spray System after a Loss of Coolant Accident because of Construction and Protective Coating Deficiencies and Foreign Material in Containment	6.8, 6.5.2
98-03	NMSS Licensees' and Certificate Holders' Year 2000 Readiness Program	N/A <sup>3)</sup>
98-02	Loss of Reactor Coolant Inventory and Associated Potential for Loss of Emergency Mitigation Functions While in a Shutdown Condition	5
98-01	Year 2000 Readiness of Computer System at Nuclear Power Plant	7
97-06	Degradation of Steam Generator Internals	5
97-05	Steam Generator Tube Inspection Techniques	5
97-04	Assurance of Sufficient Net Positive Suction Head For Emergency Core Cooling and Containment Heat Removal Pump	6
97-03	Annual Financial Surety Update Requirements for Uranium Recovery Licensees	N/A <sup>3)</sup>
97-02	Revised Contents of the Monthly Operating Report	N/A <sup>3)</sup>
97-01	Degradation of Control Rod Drive Mechanism Nozzle and Other Vessel Closure Head Penetrations	5.2.3
96-07	Interim Guidance on Transportation of Steam Generators	N/A <sup>3)</sup>
96-06 Supp. 1	Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	6
96-06	Assurance of Equipment Operability and Containment Integrity during Design-Basis Accident Conditions	6
96-05	Periodic Verification of Design-Basis Capability of Safety-Related Motor-Operated Valves	3.9.6
96-04	Boraflex Degradation in Spent Fuel Pool Storage Racks	N/A <sup>3)</sup>
96-03	Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits	운영기술 지침서



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표 1.8-2 (20 중 3)

번 호	제 목	비 고
96-02	Reconsideration of Nuclear Power Plant Security Requirements Associated with an Internal Threat	13
96-01	Testing of Safety-Related Logic Circuits	7
95-10	Relocation of Selected Technical Specifications Requirements Related to Instrumentation	운영기술 지침서
95-09	Monitoring and Training of Shippers and Carriers of Radioactive Materials	N/A <sup>3)</sup>
95-08	10 CFR 50.54(p) Process for Changes to Security Plans without Prior NRC Approval	N/A <sup>3)</sup>
95-07	Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves	3.9.6
95-06	Changes in the Operator Licensing Program	N/A <sup>3)</sup>
95-05	Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking	N/A <sup>1)</sup>
95-04	Final Disposition of the Systematic Evaluation Program Lessons-Learned Issues	N/A <sup>3)</sup>
95-03	Circumferential Cracking of Steam Generator Tubes	5.4.2
95-02	Use of NUMARC/EPRI Report TR-102348, "Guidelines on Licensing Digital Upgrades," in Determining the Acceptability of Performing Analog-to-Digital Replacements under 10 CFR 50.59	N/A <sup>3)</sup>
95-01	NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities	N/A <sup>1)</sup>
94-04	Voluntary Reporting of Additional Occupational Radiation Exposure Data	N/A <sup>2)</sup>
94-03	Intergranular Stress Corrosion Cracking of Core Shrouds in Boiling Water Reactor	N/A <sup>1)</sup>
94-02	Long-Term Solutions and Upgrade of Interim Operating Recommendations for Thermal-Hydraulic Instabilities in Boiling Water Reactors	N/A <sup>1)</sup>
94-01	Removal of Accelerated Testing and Special Reporting Requirements for Emergency Diesel Generator	운영기술 지침서
93-08	Relocation of Technical Specification Tables of Instrument Response Time Limits	운영기술 지침서
93-07	Modification of the Technical Specification Administrative Control Requirements for Emergency and Security Plans	운영기술 지침서





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표 1.8-2 (20 중 4)

번 호	제 목	비 고
93-06	Research Results on Generic Safety Issue 106, "Piping and the Use of Highly Combustible Gases in Vital Areas"	9.5.1, 9.5.9
93-05	Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing during Power Operation	운영기술 지침서
93-04	Rod Control System Failure and Withdrawal of Rod Control Cluster Assemblies, 10 CFR 50.54(f)	N/A <sup>1)</sup>
93-03	Verification of Plant Records	N/A <sup>2)</sup>
93-02	NRC Public Workshop on Commercial Grade Procurement and Dedication	N/A <sup>3)</sup>
93-01	Emergency Response Data System Test Program	N/A <sup>2)</sup>
92-09	Limited Participation by NRC in the IAEA International Nuclear Event Scale	N/A <sup>2)</sup>
92-08 Supp. 1	Fire Endurance Test Acceptance Criteria for Fire Barrier Systems Used to Separate Redundant Safe Shutdown Trains within the Same Fire Area [Draft]	N/A <sup>1)</sup>
92-08	Thermo-Lag 330-1 Fire Barriers	N/A <sup>1)</sup>
92-07	Office of Nuclear Reactor Regulation Recognition	N/A <sup>3)</sup>
92-06	Operator Licensing National Examination Schedule	N/A <sup>2)</sup>
92-05	NRC Workshop on the Systematic Assessment of Licensee Performance [SALP] Program	N/A <sup>2)</sup>
92-04	Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(f)	N/A <sup>1)</sup>
92-03	Compilation of the Current Licensing Basis : Request for Voluntary Participation in Pilot Program	N/A <sup>3)</sup>
92-02	Resolution of Generic Issue 79, "Unanalyzed Reactor Vessel [PWR] Thermal Stress during Natural Convection Cooldown"	N/A <sup>3)</sup>
92-01 Rev. 1 Supp. 1	Reactor Vessel Structural Integrity, 10 CFR 50.54(f)	N/A <sup>3)</sup>
92-01 Rev. 1	Reactor Vessel Structural Integrity, 10 CFR 50.54(f)	N/A <sup>3)</sup>
91-19	Information to Addressees Regarding New Telephone Numbers for NRC Offices Located in One White Flint North	N/A <sup>3)</sup>
91-18	Information to Licensees Regarding Two NRC Inspection Manual Sections on Resolution of Degraded and Nonconforming Conditions and on Operability	N/A <sup>3)</sup>



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표 1.8-2 (20 중 5)

번 호	제 목	비 고
91-17	Generic Safety Issue 29, "Bolting Degradation or Failures in Nuclear Power Plants"	3.12
91-16	Licensed Operators' and Other Nuclear Facility Personnel Fitness for Duty	N/A <sup>2)</sup>
91-15	Operating Experience Feedback Report, Solenoid-Operated Valve Problems at U.S. Reactors	3.9.3.2
91-14	Emergency Telecommunications	N/A <sup>3)</sup>
91-13	Request for Information Related to the Resolution of Generic Issue 130, "Essential Service Water System Failures at Multi-Unit Sites," Pursuant to 10 CFR 50.54(f)	N/A <sup>3)</sup>
91-12	Operator Licensing National Examination Schedule	N/A <sup>2)</sup>
91-11	Resolution of Generic Issues 48, "LCOs for Class 1E Vital Instrument Buses," and 49, "Interlocks and LCOs for Class 1E Tie Breakers" Pursuant to 10 CFR 50.54(f)	8.3.1.2, 8.3.2.1
91-10	Explosives Searches at Protected Area Portals	N/A <sup>2)</sup>
91-09	Modification of Surveillance Interval for the Electrical Protective Assemblies in Power Supplies for the Reactor Protection System	운영기술 지침서
91-08	Removal of Component Lists from Technical Specifications	운영기술 지침서
91-07	GI-23, "Reactor Coolant Pump Seal Failures" and Its Possible Effect on Station Blackout	5.4.1, 8.1.4
91-06	Resolution of Generic Issue A-30, "Adequacy of Safety-Related DC Power Supplies" Pursuant to 10 CFR 50.54(f)	8.3.2
91-05	License Commercial - Grade Procurement and Dedication Programs	N/A <sup>3)</sup>
91-04	Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle	N/A <sup>4)</sup> , 18 Month Fuel Cycle, 운영기술 지침서
91-03	Reporting of Safeguards Events	N/A <sup>3)</sup>
91-02	Reporting Mishaps Involving LLW Forms Prepared for Disposal	N/A <sup>2)</sup>
91-01	Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimen from Technical Specifications	10.7, 운영기술 지침서



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표 1.8-2 (20 중 6)

번 호	제 목	비 고
90-09	Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions	N/A <sup>4)</sup> , Inspection Intervals Based on 18 Month Refueling Cycle
90-08	Simulation Facility Exemptions	N/A <sup>2)</sup>
90-07	Operator Licensing National Examination Schedule	N/A <sup>2)</sup>
90-06	Resolution of Generic Issue 70, "Power Operated Relief Valve and Block Valve Reliability", and Generic Issue 94, "Additional Low-Temperature Overpressure Protection for Light-Water Reactors", Pursuant to 10 CFR 50.54(f)	5.2.2.10, 5.4.13,
90-05	Guidance for Performing Temporary Non-code Repair of ASME Code Class 1, 2 and 3 Piping	N/A <sup>2)</sup>
90-04	Request for Information on the Status of License Implementation of Generic Safety Issues Resolved with Imposition of Requirements or Corrective Actions	N/A <sup>3)</sup>
90-03 Supp. 1	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2, "Vendor Interface for Safety Related Components"	17
90-03	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2, "Vendor Interface for Safety Related Components"	17
90-02 Supp. 1	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	N/A <sup>4)</sup>
90-02	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	운영기술 지침서
90-01	Request for Voluntary Participation in NRC Regulatory Impact Survey	N/A <sup>3)</sup>
89-23	NRC Staff Response to Questions Pertaining to Implementation of 10 CFR 26	N/A <sup>3)</sup>
89-22	Potential for Increased Roof Loads and Plant Area Flood Runoff Depth at Licensed Nuclear Power Plants Due to Recent Change in Probable Maximum Precipitation Criteria Developed by the National Weather Service	2.4
89-21	Request for Information Concerning Status of Implementation of Unresolved Safety Issue(USI) Requirements	N/A <sup>3)</sup>
89-20	Protected Area Long-Term Housekeeping	N/A <sup>2)</sup>



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표 1.8-2 (20 중 7)

번 호	제 목	비 고
89-19	Request for Action Related to Resolution of Unresolved Safety Issue A-47 "Safety Implication of Control System in LWR Nuclear Power Plant" Pursuant to 10 CFR 50.54(f)	10.4
89-18	Resolution of Unresolved Safety Issue A-17, "Systems Interactions in Nuclear Power Plants"	N/A <sup>1)</sup>
89-17	Planned Administrative Changes to the NRC Operator Licensing Written Examination Process	N/A <sup>2)</sup>
89-16	Installation of a Hardened Wet Well Vent	N/A <sup>1)</sup>
89-15	Emergency Response Data System	N/A <sup>2)</sup>
89-14	Line-Item Improvements in Technical Specifications - Removal of 3.25 Limit on Extending Surveillance Intervals	운영기술 지침서 SR 3.0.2
89-13 Supp. 1	Service Water System Problems Affecting Safety-Related Equipment	9.2.1.2 9.2.1.4
89-13	Service Water System Problems Affecting Safety-Related Equipment	9.2.1
89-12	Operator Licensing Examinations	N/A <sup>2)</sup>
89-11	Resolution of Generic Issue 101 "Boiling Water Reactor Water Level Redundancy"	N/A <sup>1)</sup>
89-10 Supp. 6	Information on Schedule and Grouping, and Staff Responses to Additional Public Questions	N/A <sup>2)</sup>
89-10 Supp. 5	Inaccuracy of Motor-Operated Valve Diagnostic Equipment	N/A <sup>1)</sup>
89-10 Supp. 4	Consideration of Valve Mispositioning in BWRs	N/A <sup>1)</sup>
89-10 Supp. 3	Consideration of the Results of NRC-Sponsored Tests of Motor-Operated Valves	N/A <sup>1)</sup>
89-10 Supp. 2	Availability of Program Descriptions	N/A <sup>3)</sup>
89-10 Supp. 1	Results of the Public Workshops	3.9.6.2
89-10	Safety-Related Motor-Operated Valve Testing and Surveillance	3.9.6.2
89-09	ASME Section III Components Replacements	N/A <sup>2)</sup>
89-08	Erosion/Corrosion - Induced Pipe Wall Thinning	3.6
89-07 Supp. 1	Power Reactor Safeguards Contingency Planning for Surface Vehicle Bombs	N/A <sup>2)</sup>



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표 1.8-2 (20 중 8)

번 호	제 목	비 고
89-07	Power Reactor Safeguards Contingency Planning for Surface Vehicle Bombs	13
89-06	Task Action Plan Item I.D.2 - Safety Parameter Display System - 10 CFR 50.54(f)	18
89-05	Pilot Testing of the Fundamentals Examination	N/A <sup>2)</sup>
89-04 Supp. 1	Guidance on Developing Acceptable Inservice Testing Programs	6.6
89-04	Guidance on Developing Acceptable Inservice Testing Programs	6.6
89-03	Operator Licensing National Examination Schedule	N/A <sup>2)</sup>
89-02	Actions to Improve the Detection of Counterfeit and Fraudulently Marketed Products	N/A <sup>2)</sup>
89-01	Implementation of Programmatic Controls for Radiological Effluent Technical Specifications in the Administrative Controls Section of the Technical Specification and the Relocation of Procedural Details of RETS to the Offsite Dose Calculation Manual or to the Process Control Program	운영기술 지침서
88-20 Supp. 5	Individual Plant Examination of External Events for Severe Accident Vulnerabilities	N/A <sup>5)</sup>
88-20 Supp. 4	Individual Plant Examination of External Events(IPEEE) for Severe Accident Vulnerabilities	N/A <sup>5)</sup>
88-20 Supp. 3	Completion of Containment Performance Improvement Program and Forwarding of Insights for Use in the Individual Plant Examination for Severe Accident Vulnerabilities	N/A <sup>5)</sup>
88-20 Supp. 2	Accident Management Strategies for Consideration in the Individual Plant Examination Process	N/A <sup>5)</sup>
88-20 Supp. 1	Initiation of the Individual Plant Examination for Severe Accident Vulnerabilities - 10 CFR 50.54(f)	N/A <sup>5)</sup>
88-20	Individual Plant Examination for Severe Accident Vulnerabilities	N/A <sup>5)</sup>
88-19	Use of Deadly Force by License Guards to Prevent Theft of Special Nuclear Material	N/A <sup>2)</sup>
88-18	Plant Record Storage on Optical Disks	N/A <sup>2)</sup>
88-17	Loss of Decay Heat Removal	5.4.7, 7.7.1
88-16	Removal of Cycle-Specific Parameter Limits from Technical Specifications	운영기술 지침서



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표 1.8-2 (20 중 9)

번 호	제 목	비 고
88-15	Electric Power Systems-Inadequate Control over Design Process	N/A <sup>5)</sup>
88-14	Instrument Air Supply System Problems Affecting Safety-Related Equipment	9.3.1
88-13	Operator Licensing Examination	N/A <sup>2)</sup>
88-12	Removal of Fire Protection Requirements from Technical Specifications	운영기술 지침서
88-11	NRC Position on Radiation Embrittlement of Reactor Vessel Materials and its Impact on Plant Operations	5.3.1
88-10	Purchase of GSA Approved Security Containers	N/A <sup>2)</sup>
88-09	Pilot Testing of Fundamentals Examinations	N/A <sup>2)</sup>
88-08	Mail Sent or Delivered to the Office of Nuclear Reactor Regulation	N/A <sup>3)</sup>
88-07	Modified Enforcement Policy Relation to 10 CFR 50.49, "Environmental Qualification of Electrical Equipment Important to Safety for Nuclear Power Plants"	N/A <sup>3)</sup>
88-06	Removal of Organization Charts from Technical Specification Administrative Control Requirements	N/A <sup>4)</sup>
88-05	Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR Plants	5.2.3
88-04	Distribution of Gems Irradiated in Research Reactors	N/A <sup>1)</sup>
88-03	Resolution of Generic Safety Issue 93, "Steam Bindings of Auxiliary Feedwater Pumps"	10.4.9
88-02	Integrated Safety Assessment Program II(ISAP II)	N/A <sup>5)</sup>
88-01 Supp. 1	NRC Position on Intergranular Stress Corrosion Cracking in BWR Austenitic Stainless Steel Piping	N/A <sup>1)</sup>
88-01	NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping	N/A <sup>1)</sup>
87-16	Transmittal of NUREG-1262, "Answers to Questions at Public Meeting Regarding Implementation of 10 CFR 55 on Operators' Licenses"	N/A <sup>3)</sup>
87-15	Policy Statement on Deferred Plants	N/A <sup>2)</sup>
87-14	Operator Licensing Examinations	N/A <sup>2)</sup>
87-13	Integrity of Requalification Examinations at Non-Power Reactor	N/A <sup>1)</sup>
87-12	Loss of Residual Heat Removal While the Reactor Coolant System is Partially Filled	GL 88-17 참조

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표 1.8-2 (20 중 10)

번 호	제 목	비 고
87-11	Relaxation in Arbitrary Intermediate Pipe Rupture Requirements	3.6.2
87-10	Implementation of 10 CFR 73.57, Requirements for FBI Criminal History Checks	N/A <sup>2)</sup>
87-09	Sections 3.0 and 4.0 of the Standard Technical Specifications on the Applicability of Limiting Conditions for Operation and Surveillance Requirements	운영기술 지침서
87-08	Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements	N/A <sup>2)</sup>
87-07	Information Transmittal of Final Rulemaking for Revisions to Operator Licensing - 10 CFR 55 and Conforming Amendments	N/A <sup>2)</sup>
87-06	Periodic Verification of Leak Tight Integrity of Pressure Isolation Valves	운영기술 지침서
87-05	Request for Additional Information - Assessment of License Measures to Mitigate and /or Identify Potential Degradation of Mark I Drywells	N/A <sup>1)</sup>
87-04	Temporary Exemption from Provisions of the FBI Criminal History Rule for Temporary Workers	N/A <sup>2)</sup>
87-03	Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operation Reactors, Unresolved Safety Issue A-46	3.10
87-02 Supp. 1	Supplemental Safety Evaluation Report No. 2 on SQUG Generic Implementation Procedure, Rev.2	N/A <sup>2)</sup>
87-02	Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operation Reactors, USI A-46	3.10
87-01	Public Availability of the NRC Operator Licensing Examination Question Bank	N/A <sup>2)</sup>
86-17	Availability of NUREG-1169, "Technical Findings Related to Generic Issue C-8; BWR Main Steam Isolation Valve Leakage and Leakage Treatment Methods"	N/A <sup>1)</sup>
86-16	Westinghouse ECCS Evaluation Models	N/A <sup>1)</sup>
86-15	Information Relating to Compliance with 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants."	GL 88-07 참조
86-14	Operator Licensing Examinations	N/A <sup>2)</sup>
86-13	Potential Inconsistency between Safety Analyses and Technical Specifications	운영기술 지침서



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표 1.8-2 (20 중 11)

번 호	제 목	비 고
86-12	Criteria for Unique Purpose Exemption from Conversion from the Use of HEU Fuel	N/A <sup>3)</sup>
86-11	Distribution of Products Irradiated in Research Reactors	N/A <sup>1)</sup>
86-10 Supp. 1	Fire Endurance Test Acceptance Criteria for Fire Barrier Systems Used to Separate Redundant Safe Shutdown Trains Within the Same Fire Area	9.5.1
86-10	Implementation of Fire Protection Requirements	9.5.1
86-09	Technical Resolution of Generic Issue No. B-59-(N-1) Loop Operation in BWRs and PWRs	N/A <sup>4)</sup>
86-08	Availability of Supplement 4 to NUREG-0933, "A Prioritization of Generic Safety Issues"	N/A <sup>3)</sup>
86-07	Transmittal of NUREG-1190 Regarding the San Onofre Unit 1 Loss of Power and Water Hammer Event	N/A <sup>3)</sup>
86-06	Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps"	N/A <sup>2)</sup>
86-05	Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps"[BWR]	N/A <sup>1)</sup>
86-04	Policy Statement on Engineering Expertise on Shift	N/A <sup>2)</sup>
86-03	Applications for License Amendments	N/A <sup>2)</sup>
86-02	Technical Resolution of Generic Issue B-19 - Thermal Hydraulic Stability [BWR]	N/A <sup>1)</sup>
86-01	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	N/A <sup>1)</sup>
85-22	Potential for Loss of Post-LOCA Recirculation Capability due to Insulation Debris Blockage	N/A <sup>3)</sup>
85-21	Not issued.	-
85-20	Resolution of Generic Issue 69 [B&W]	N/A <sup>1)</sup>
85-19	Reporting Requirements on Primary Coolant Iodine Spikes	운영기술 지침서
85-18	Operating Licensing Examinations	N/A <sup>2)</sup>
85-17	Availability of Supplement 2 and 3 to NUREG-0933	GL 86-08 참조
85-16	High Boron Concentrations	N/A <sup>1)</sup>
85-15	Information Relating to the Deadlines for Compliance with 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plant"	GL 88-07 참조





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표 1.8-2 (20 중 12)

번 호	제 목	비 고
85-14	Commercial Storage at Power Reactor Sites of Low-Level Radioactive Waste Not Generated by the Utility	N/A <sup>2)</sup>
85-13	Transmittal of NUGEG-1154 Regarding the Davis-Besse Loss of Main and Auxiliary Feedwater Event	N/A <sup>3)</sup>
85-12	Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" [Westinghouse]	N/A <sup>1)</sup>
85-11	Completion of Phase II of "Control of Heavy Loads at Nuclear Power Plants" NUREG-0612	9.1.4
85-10	Technical Specifications for Generic Letter 83-28, Items 4.3 and 4.4 [B&W]	N/A <sup>1)</sup>
85-09	Technical Specifications for Generic Letter 83-28, Item 4.3(Westinghouse NSSS)	N/A <sup>1)</sup>
85-08 Addenda	Revision of NRC Form 439, "Report of Terminating Individual's Occupational Exposure"	N/A <sup>2)</sup>
85-08	10 CFR 20.408 Termination Reports - Format	N/A <sup>2)</sup>
85-07	Implementation of Integrated Schedules for Plant Modifications	N/A <sup>2)</sup>
85-06	Quality Assurance Guidance for ATWS Equipment that is not Safety-Related	N/A <sup>3)</sup>
85-05	Inadvertent Boron Dilution Events	15.4.6
85-04	Operator Licensing Examinations	N/A <sup>2)</sup>
85-03	Clarification of Equivalent Control Capacity for Standby Liquid Control Systems [BWR]	N/A <sup>1)</sup>
85-02	Staff Recommended Actions Stemming from NRC Integrated Program for the Resolution of Unresolved Safety Issues Regarding Steam Generator Tube Integrity	5.4.2, 7.7.1.6
85-01	Fire Protection Policy Steering Committee Report	9.5.1
84-24	Certification of Compliance to 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants"	3.10
84-23	Reactor Vessel Water Level Instrumentation in BWRs	N/A <sup>1)</sup>
84-22	Not Issued.	
84-21	Long Term Low Power Operation in Pressurized Water Reactors	N/A <sup>2)</sup>
84-20	Scheduling Guidance for License Submittals of Reloads that Involve Unreviewed Safety Questions	N/A <sup>3)</sup>



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표 1.8-2 (20 중 13)

번 호	제 목	비 고
84-19	Availability of Supplement 1 to NUREG-0933, "A Prioritization of Generic Safety Issues"	GL 85-17 참조
84-18	Filing of Applications for Licenses and Amendments	N/A <sup>3)</sup>
84-17	Annual Meeting to Discuss Recent Development Regarding Operator Training, Qualifications, and Examinations	N/A <sup>2)</sup>
84-16	Adequacy of On-Shift Operating Experience for Near Term Operating License Applicants	N/A <sup>2)</sup>
84-15	Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability	8.3.1.1
84-14	Replacement and Requalification Training Program	N/A <sup>2)</sup>
84-13	Technical Specification for Snubbers	운영기술 지침서
84-12	Compliance with 10 CFR 61 and Implementation of the Radiological Effluent Technical Specifications and Attendant Process Control Program	N/A <sup>2)</sup>
84-11	Inspections of BWR Stainless Steel Piping	N/A <sup>1)</sup>
84-10	Administration of Operating Tests Prior to Initial Criticality	N/A <sup>2)</sup>
84-09	Recombiner Capability Requirements of 10 CFR 50.44(c)(3)(II)	6.2.5
84-08	Interim Procedures for NRC Management of Plant-Specific Backfitting	N/A <sup>3)</sup>
84-07	Procedural Guidance for Pipe Replacement at BWRs	N/A <sup>1)</sup>
84-06	Operator and Senior Operator License Examination Criteria for Passing Grade	N/A <sup>2)</sup>
84-05	Change to NUREG-1021, "Operator Licensing Examiner Standards"	N/A <sup>2)</sup>
84-04	Safety Evaluation of Westinghouse Topical Reports Dealing with Elimination of Postulated Pipe Breaks in PWR Primary Main Loops	N/A <sup>4)</sup>
84-03	Availability of NUREG-0933, "A Prioritization of Generic Safety Issues"	GL 84-19 참조
84-02	Notice of Meeting Regarding Facility Staffing	N/A <sup>2)</sup>
84-01	NRC Use of the Terms, "Important to Safety" and "Safety Related"	N/A <sup>3)</sup>
83-44	Availability of NUREG-1021, "Operator Licensing Examiner Standards"	N/A <sup>2)</sup>
83-43	Reporting Requirements of 10 CFR 50.72 and 50.73, and Standard Technical Specifications	N/A <sup>3)</sup>

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표 1.8-2 (20 중 14)

번 호	제 목	비 고
83-42	Clarification to Generic Letter 81-07 Regarding Response to NUREG-0612, "Control of Heavy Loads at Nuclear Power Plant"	9.1.4
83-41	Fast Cold Starts of Diesel Generators	N/A <sup>3)</sup>
83-40	Operator Licensing Examination	N/A <sup>2)</sup>
83-39	Voluntary Survey of Licensed Operators	N/A <sup>2)</sup>
83-38	NUREG-0965, "NRC Inventory of Dams"	N/A <sup>3)</sup>
83-37	NUREG-0737 Technical Specifications	운영기술 지침서
83-36	NUREG-0737 Technical Specifications [BWR]	N/A <sup>1)</sup>
83-35	Clarification of TMI Action Plan Item II.K.3.31	6.3.3
83-34	Not Issued.	-
83-33	NRC Positions on Certain Requirements of Appendix R to 10 CFR 50	9.5.1
83-32	NRC Staff Recommendations Regarding Operator Action for Reactor Trip and ATWS	N/A <sup>2)</sup>
83-31	Safety Evaluation of "Abnormal Transient Operating Guidelines" [BWR]	N/A <sup>1)</sup>
83-30	Deletion of Standard Technical Specification Surveillance Requirement 4.8.1.1.2.d.6 for Diesel Generator Testing	N/A <sup>3)</sup>
83-29	Not Issued.	-
83-28 Supp. 1	Required Actions Based on Generic Implication of Salem ATWS Events	N/A <sup>3)</sup>
83-28	Required Actions Based on Generic Implication of Salem ATWS Events	4.1
83-27	Surveillance Intervals in Standard Technical Specifications	GL 91-04 참조
83-26	Clarification of Surveillance Requirements for Diesel Fuel Impurity Level Tests	N/A <sup>3)</sup>
83-25	Not Issued	-
83-24	TMI Task Action Plan Item I.G.1, "Special Low Power Testing and Training", Recommendations for BWRs	N/A <sup>1)</sup>
83-23	Safety Evaluation of "Emergency Response Guidelines" [C-E]	N/A <sup>3)</sup>
83-22	Safety Evaluation of "Emergency Response Guidelines" [Westinghouse]	N/A <sup>1)</sup>



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표 1.8-2 (20 중 15)

번 호	제 목	비 고
83-21	Clarification of Access Control Procedures for Law Enforcement Visits	N/A <sup>3)</sup>
83-20	Integrated Scheduling for Implementation of Plant Modifications	N/A <sup>3)</sup>
83-19	New Procedures for Providing Public Notice Concerning Issuance of Amendment to Operating Licenses	N/A <sup>3)</sup>
83-18	NRC Staff Review of the BWR Owner's Group Control Room Survey Program	N/A <sup>1)</sup>
83-17	Integrity of the Requalification Examination for Renewal of Reactor Operator and Senior Reactor Operator Licenses	N/A <sup>2)</sup>
83-16	Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit No. 1	N/A <sup>3)</sup>
83-15	Implementation of Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations, Rev. 1"	N/A <sup>3)</sup>
83-14	Definition of "Key Maintenance Personnel"	N/A <sup>2)</sup>
83-13	Clarification of Surveillance Requirements for HEPA Filters and Charcoal Absorber Units in Standard Technical Specifications on ESF Cleanup Systems	운영기술 지침서
83-12A	Issuance of NRC Form 398 - Personal Qualifications Statement -Licensee	N/A <sup>3)</sup>
83-12	Issuance of NRC Form 398 - Personal Qualifications Statement -Licensee	N/A <sup>3)</sup>
83-11	License Qualification for Performing Safety Analyses in Support of Licensing Actions	N/A <sup>3)</sup>
83-10f	Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" [B&W NSSS]	N/A <sup>1)</sup>
83-10e	Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" [B&W NSSS]	N/A <sup>1)</sup>
83-10d	Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" [W NSSS]	N/A <sup>1)</sup>
83-10c	Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" [W NSSS]	N/A <sup>1)</sup>
83-10b	Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" [C-E NSSS]	GL 86-06 참조
83-10a	Resolution of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" [C-E NSSS]	GL 86-06 참조

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표 1.8-2 (20 중 16)

번 호	제 목	비 고
83-09	Review of Combustion Engineering Owner's Group Emergency Procedures Guideline Program	GL 86-06 참조
83-08	Modification of Vacuum Breakers on Mark I Containments	N/A <sup>1)</sup>
83-07	The Nuclear Waste Policy Act of 1982	N/A <sup>3)</sup>
83-06	Certificates and Revised Format for Reactor Operator and Senior Reactor Operator Licenses	N/A <sup>2)</sup>
83-05	Safety Evaluation of "Emergency Procedure Guidelines, Rev.2", NEDO-24934, June 1982 [BWRs]	N/A <sup>1)</sup>
83-04	Regional Workshops Regarding Supplement 1 to NUREG-0737, Requirements for Emergency Response Capability	N/A <sup>3)</sup>
83-03	Not Issued.	-
83-02	NUREG-0737 Technical Specifications [BWRs]	N/A <sup>1)</sup>
83-01	Operator Licensing Examination Site Visit	N/A <sup>2)</sup>
82-39	Problems with the Submittals of 10 CFR 72.21 Safeguards Information for Licensing Review	N/A <sup>3)</sup>
82-38	Meeting to Discuss Recent Developments for Operating Licensing Examinations	N/A <sup>2)</sup>
82-34/37	Not Issued.	-
82-33	Supplement 1 to NUREG-0737 - Requirements for Emergency Response Capability	7.5
82-32	Potential Steam Generator Related Generated Requirements	N/A <sup>3)</sup>
82-31	Not Issued.	-
82-30	Filing Relating to 10 CFR 50 Production and Utilization Facilities	N/A <sup>3)</sup>
82-29	Not Issued.	-
82-28	Inadequate Core Cooling Instrumentation System	6.3.5, 7.5
82-27	Transmittal of NUREG-0763 and NUREG-0783 [BWRs]	N/A <sup>1)</sup>
82-26	NUREG-0744 Rev. 1 - Pressure Vessel Material Fracture Toughness	5.3.1
82-25	Integrated IAEA Exercise for Physical Inventory at LWRs	N/A <sup>3)</sup>
82-24	Safety/Relief Valve Quencher Loads: Evaluation for BWR Mark II and III Containments	N/A <sup>1)</sup>
82-23	Inconsistency between Requirements of 10 CFR 73.40(d) and Standard Technical Specifications for Performing Audits of Safeguards Contingency Plans	N/A <sup>3)</sup>



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표 1.8-2 (20 중 17)

번 호	제 목	비 고
82-22	Congressional Request for Information concerning Steam Generator Tube Integrity	N/A <sup>2)</sup>
82-21	Technical Specifications for Fire Protection Audits	N/A <sup>2)</sup>
82-20	Guidance for Implementing Standard Review Plan Rule	N/A <sup>3)</sup>
82-19	Submittal of Copies of Document to NRC	N/A <sup>3)</sup>
82-18	Reactor Operator and Senior Reactor Operator Requalification Examinations	N/A <sup>2)</sup>
82-17	Inconsistency Between Requirements of 10 CFR 50.54(t) and Standard Technical Specifications for Performing Audits of Emergency Preparedness Programs	N/A <sup>2)</sup>
82-16	NUREG-0737 Technical Specifications	N/A <sup>3)</sup>
82-15	Not Issued.	-
82-14	Submittal of Documents to the Nuclear Regulatory Commission	N/A <sup>3)</sup>
82-13	Reactor Operator and Senior Reactor Operator Examinations	N/A <sup>2)</sup>
82-12	Nuclear Power Plant Staff Working Hours	N/A <sup>2)</sup>
82-11	Transmittal of NUREG-0916 Relative to the Restart of R. E. Ginna Nuclear Power Plant	N/A <sup>3)</sup>
82-10	Post-TMI Requirements	N/A <sup>2)</sup>
82-09	Environmental Qualification of Safety-Related Electrical Equipment	N/A <sup>3)</sup>
82-08	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	N/A <sup>3)</sup>
82-07	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	N/A <sup>3)</sup>
82-06	Not Issued.	-
82-05	Post-TMI Requirements	GL 82-10 참조
82-04	Use of INPO SEE-IN Program	N/A <sup>3)</sup>
82-03	High Burnup MAPLHGR Limits [BWRs]	N/A <sup>1)</sup>
82-02	Nuclear Power Plant Staff Working Hours	N/A <sup>2)</sup>
82-01	New Applications Survey	N/A <sup>3)</sup>
81-40	Qualifications of Reactor Operators	N/A <sup>2)</sup>
81-39	NRC Volume Reduction Policy	N/A <sup>3)</sup>
81-38	Storage of Low Level Radioactive Wastes at Power Reactor Site	N/A <sup>2)</sup>



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표 1.8-2 (20 중 18)

번 호	제 목	비 고
81-37	ODYN Code Reanalysis Requirements [BWRs]	N/A <sup>1)</sup>
81-36	Revised Schedule for Completion of TMI Action Plan Item II.D.1, Relief and Safety Valve Testing	5.4.13
81-35	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	N/A <sup>1)</sup>
81-34	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	N/A <sup>1)</sup>
81-33	Not Issued.	-
81-32	NUREG-0737, Item II.K.3.44 - Evaluation of Anticipated Transients Combined with Single Failure [BWRs]	N/A <sup>1)</sup>
81-31	Not Issued.	-
81-30	Safety Concerns Associated with pipe Breaks in the BWR Scram System	N/A <sup>1)</sup>
81-29	Simulator Examinations	N/A <sup>2)</sup>
81-28	Steam Generator Overfill	N/A <sup>2)</sup>
81-27	Privacy and Proprietary Material in Emergency Plans	N/A <sup>2)</sup>
81-26	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	N/A <sup>1)</sup>
81-25	Change in Implementing Schedule for Submission and Evaluation of Upgraded Emergency Plans	N/A <sup>2)</sup>
81-24	Multi-Plant Issue B-56 Control Rods Fail to Fully Insert [BWRs]	N/A <sup>1)</sup>
81-23A	INPO Evaluation Reports	N/A <sup>3)</sup>
81-23	INPO Plant Specific Evaluation Report	N/A <sup>3)</sup>
81-22	Engineering Evaluation of the H.B.Robinson Reactor Coolant System Leak on 1/29/81	N/A <sup>3)</sup>
81-21	Natural Circulation Cooldown	5.4.1
81-20	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	N/A <sup>1)</sup>
81-19	Thermal Shock to Reactor Pressure Vessels	5.2.2.11
81-18	BWR Scram Discharge System - Clarification of Diverse Instrumentation Requirements	N/A <sup>1)</sup>
81-17	Functional Criteria for Emergency Response Facilities	N/A <sup>2)</sup>
81-16	NUREG-0737 Item I.C.1 SER on Abnormal Transient Operating Guidelines [B&W]	N/A <sup>1)</sup>



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표 1.8-2 (20 중 19)

번 호	제 목	비 고
81-15	Environmental Qualification of Class 1E Electrical Equipment - Clarification of Staffs Handling of Proprietary Information	N/A <sup>3)</sup>
81-14	Seismic Qualification for Auxiliary Feedwater System	10.4.9
81-13	SER for GEXL Correlation for 8x8R Fuel Reload Applications	N/A <sup>1)</sup>
81-12, R1	Fire Protection Rule	9.5.1.3.6
81-11	BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking	N/A <sup>1)</sup>
81-10	Post-TMI Requirements for the Emergency Operations Facility	N/A <sup>2)</sup>
81-09	BWR Scram Discharge System	N/A <sup>1)</sup>
81-08	ODYN Code [BWRs]	N/A <sup>1)</sup>
81-07	Control of Heavy Loads	9.1.4
81-06	Periodic Updating of Final Safety Analysis Reports	N/A <sup>2)</sup>
81-05	Information Regarding the Program for Environmental Qualification of Safety-Related Electrical Equipment	N/A <sup>1)</sup>
81-04	Emergency Procedures and Training for Station Blackout Events	N/A <sup>2)</sup>
81-03	Implementation of NUREG-0313[BWRs]	N/A <sup>1)</sup>
81-02	Analysis, Conclusion and Recommendation Concerning Operator Licensing	N/A <sup>2)</sup>
81-01	Qualification of Inspection, Examination and Audit Personnel	N/A <sup>2)</sup>
80-109	Guidelines for SEP Soil-Structure Interaction Reviews	3.7 부록B
80-106	Report on ECCS Cladding Models, NUREG-0630	GL 80-01 기술됨
80-99	Technical Specifications Revisions for Snubber Surveillance	운영기술 지침서
80-35	Effect of a DC Power Supply Failure on ECCS Performance	8.3.2, 15.3
80-30	Clarifications of the Term "Operable" as It Applies to the Single-Failure Criterion for Safety Systems Required by Technical Specifications	운영기술 지침서
80-19	Resolution of Enhanced Fission Gas Release Concern	1.6, 4.2.1, 4.2.3

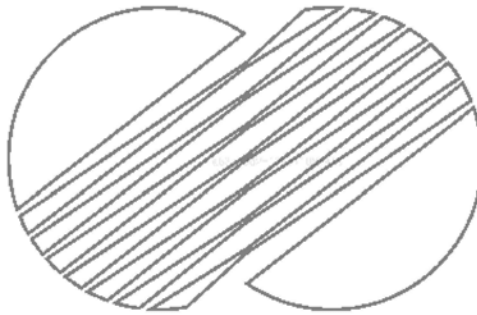


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표 1.8-2 (20 중 20)

번 호	제 목	비 고
80-01	NUREG-0630, "Cladding, Swelling, and Rupture - Models for LOCA Analysis"	1.6, 4.2.3, 6



주 : 적용하지 않는 항목(N/A)에 대한 예외기준

- 1) 특정 원자로형이나 특정 공급사에 해당되는 항목
- 2) 발전소 운영 및 특정 발전소 설계에 해당되는 항목
- 3) 설계요건에 포함되지 않는 항목
- 4) 강제적인 사항은 아니지만 인허가 신청자가 대체방안으로 채택하는 항목
- 5) 별도보고서인 “확률론적안전성평가보고서”에 기술되는 항목



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표 1.8-3 (5 중 1)

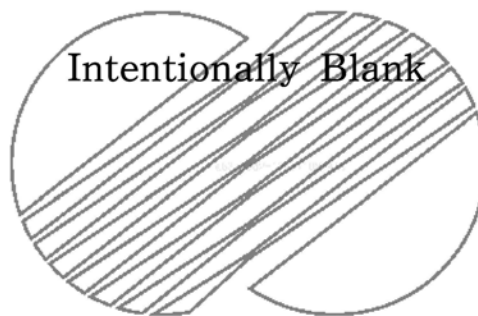
신고리 3,4호기에 대한 NRC Bulletins의 적용성 분석

번 호	제 목	비 고
07-01	Security Officer Attentiveness	N/A <sup>4)</sup>
05-02	Emergency Preparedness and response Actions for Security-Based Events	N/A <sup>4)</sup>
04-01	Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at Pressurized-Water Reactors	N/A <sup>4)</sup>
03-02	Leakage from Reactor Pressure Vessel Lower Head Penetrations and Reactor Coolant Pressure Boundary Integrity	N/A <sup>1)</sup>
03-01	Potential Impact of Debris Blockage on Emergency Sump Recirculation at Pressurized-Water Reactors	9.1.4
02-02	Reactor Pressure Vessel Head and Vessel Head Penetration Nozzle Inspection Programs	N/A <sup>1)</sup>
02-01	Reactor Pressure Vessel Head Degradation and Reactor Coolant Pressure Boundary Integrity	N/A <sup>1)</sup>



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표 1.8-3 (5 중 2)

번 호	제 목	비 고
97-02	Puncture Testing of Shipping Packages Under 10 CFR 71	N/A <sup>4)</sup>
97-01	Potential for Erroneous Calibration, Dose Rate, or Radiation Exposure Measurements with Certain Victoreen Model 530 and 530SI Electrometer/Dosimeters	N/A <sup>4)</sup>
96-04	Chemical, Galvanic, or Other Reactions in Spent Fuel Storage and Transportation Casks	N/A <sup>4)</sup>
96-03	Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Boiling-Water Reactors	N/A <sup>1)</sup>
96-02	Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment	9.1.4
96-01	Control Rod Insertion Problems	N/A <sup>1)</sup>
95-02	Unexpected clogging of a Residual Heat Removal(RHR) Pump Strainer While Operating in Suppression Pool Cooling Mode	N/A <sup>1)</sup>
95-01	Quality Assurance Program for Transportation of Radioactive Material	N/A <sup>1)</sup>
94-02	Corrosion Problems in Certain Stainless Steel Packagings Used to Transport Uranium Hexafluoride	N/A <sup>1)</sup>
94-01	Potential Fuel Pool Draindown Caused by Inadequate Maintenance Practices at Dresden Unit 1	9.1.3, 9.4.2
93-03	Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWR	N/A <sup>1)</sup>
93-02 및 Supp. 1	Debris Plugging of Emergency Core Cooling Suction Strainers	N/A <sup>3)</sup>
93-01	Release of Patients after Brachytherapy Treatment with Remote Afterloading Devices	N/A <sup>2)</sup>
92-03	Release of Patients after Brachytherapy	N/A <sup>2)</sup>
92-02	Safety Concerns Relating to "End of Life" of Aging Theratronics Teletherapy Units	N/A <sup>2)</sup>
92-01 Supp. 1	Failure of Thermo-Lag 330 Fire Barrier System to Perform its Specified Fire Endurance Function	N/A <sup>1)</sup>
92-01	Failure of Thermo-Lag 330 Fire Barrier System to Maintain Cabling in Wide Cable Trays and Small Conduits Free from Fire Damage	N/A <sup>1)</sup>
91-01	Reporting Loss of Criticality Safety Controls	N/A <sup>1)</sup>



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신고리 3,4호기 최종안전성분석보고서

표 1.8-3 (5 중 3)

번 호	제 목	비 고
90-02	Loss of Thermal Margin Caused by Channel Box Bow	N/A <sup>1)</sup>
90-01 및 Supp. 1	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	N/A <sup>2)</sup>
89-03	Potential Loss of Required Shutdown Margin During Refueling Operation	14.1
89-02	Stress Corrosion Cracking of High-Hardness Type 410 Stainless Steel Internal Preloaded Bolting in Anchor Darling Model S350W Swing Check Valves or Valves of Similar Design	3.9.6
89-01 및 Supp. 1,2	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	N/A <sup>1)</sup>
88-11	Pressurizer Surge Line Thermal Stratification	3.9A
88-10 및 Supp. 1	Nonconforming Molded-Case Circuit Breakers	N/A <sup>3)</sup>
88-09	Thimble Tube Thinning in Westinghouse Reactors	N/A <sup>1)</sup>
88-08 및 Supp.1,2,3	Thermal Stress in Piping Connected to Reactor Coolant Systems	3.9, 3.9A
88-07 및 Supp. 1	Power Oscillations in Boiling Water Reactors	N/A <sup>1)</sup>
88-06	Actions to be Taken for the Transportation of Model No. Spec 2-T Radiographic Exposure Device	N/A <sup>2)</sup>
88-05 및 Supp. 1,2	Nonconforming Materials Supplied by Piping Suppliers, Inc.	N/A <sup>2)</sup>
88-04	Potential Safety-Related Pump Loss	5.4.7.2
88-03	Inadequate Latch Engagement in HFA Type Latching Relays Manufactured by GE Company	N/A <sup>1)</sup>
88-02	Rapidly Propagating Fatigue Cracks in Steam Generator Tubes [Westinghouse]	N/A <sup>1)</sup>
88-01	Defects in Westinghouse Circuit Breakers	N/A <sup>1)</sup>
87-02 및 Supp. 1	Fastener Testing to Determine Conformance with Applicable Material Specifications	N/A <sup>3)</sup>
87-01	Thinning of Pipe Walls in Nuclear Power Plants	3.6
86-04	Defective Teletherapy Timer that May Not Terminate Treatment Dose	N/A <sup>2)</sup>
86-03	Potential Failure of Multiple ECCS Pumps Due to Single Failure of Air-Operated Valve in Minimum Flow Recirculation Line	N/A <sup>4)</sup>



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신고리 3,4호기 최종안전성분석보고서

표 1.8-3 (5 중 4)

번 호	제 목	비 고
86-02	Static "O" Ring Differential Pressure Switches	N/A <sup>2)</sup>
86-01	Minimum Flow Logic Problems that Could Disable RHR Pumps [BWRs]	N/A <sup>1)</sup>
85-03 및 Supp. 1	Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings	3.9.6
85-02	Undervoltage Trip Attachments of Westinghouse DB-50 Type Reactor Trip Breakers	N/A <sup>1)</sup>
85-01	Steam Binding of Auxiliary Feedwater Pumps	10.4.9
84-03	Refueling Cavity Water Seal	9.1.4
84-02	Failures of General Electric Type HFA Relays in Use in Class 1E Safety Systems	N/A <sup>1)</sup>
84-01	Cracks in Boiling Water Reactor Mark I Containment Vent Headers	N/A <sup>1)</sup>
83-08	Electrical Circuit Breakers with an Undervoltage Trip Feature Used in Safety Related Applications	N/A <sup>1)</sup>
83-07 및 Supp. 1,2	Apparently Fraudulent Products Sold by Ray Miller, Inc.	N/A <sup>2)</sup>
83-06	Nonconforming Materials Supplied by Tube-Line Corporation Facilities	N/A <sup>2)</sup>
83-05	ASME Code Pumps and Spare Parts Manufactured by the Haywood Tyler Pump Company	N/A <sup>2)</sup>
83-04	Failure of Undervoltage Trip Function of Reactor Trip Breakers	N/A <sup>3)</sup>
83-03	Check Valve Failures in Raw Water Cooling Systems of Diesel Generators	3.9.6, 9.5.5
83-02	Stress Corrosion Cracking in Large-Diameter Stainless Steel Recirculation System Piping at BWR Plants	N/A <sup>1)</sup>
83-01	Failure of Trip Breakers to Open on Automatic Trip Signal	N/A <sup>3)</sup>
82-04	Deficiencies in Primary Containment Electrical Penetration Assemblies	N/A <sup>3)</sup>
82-03	Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants	N/A <sup>1)</sup>
82-02	Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of PWR Plants	5.2.3, 5.2.4
82-01 및 Rev. 1	Alteration of Radiographs of Welds in Piping Subassemblies	N/A <sup>3)</sup>



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신고리 3,4호기 최종안전성분석보고서

표 1.8-3 (5 중 5)

번 호	제 목	비 고
81-03	Flow Blockage of Cooling Water to Safety System Components	9.2.1
82-01 및 Supp. 1	Failure of Gate Type Valves to Close Against Differential Pressure	3.9.6, GL 89-10
81-01	Surveillance of Mechanical Snubbers	N/A <sup>3)</sup>
80-24	Prevention of Damage Due to Water Leakage Inside Containment	N/A <sup>3)</sup>
80-20	Failures of Westinghouse Type W-2 Spring Return to Neutral Control Switches	N/A <sup>4)</sup>
80-19	Failure of Mercury-Wetted Matrix Relays in Protection Systems of Plants Designed by C-E	N/A <sup>2)</sup>
80-18	Maintenance of Adequate Minimum Flow Through Centrifugal Charging Pumps Following Secondary-Side High- Energy Line Rupture	-
80-15	Possible Loss of Emergency Notification System with Loss of Off-Site Power	N/A <sup>4)</sup> GL 91-14
80-11	Masonry Wall Design	N/A <sup>2)</sup> , 3.8.4.6
80-08	Examination of Containment Liner Penetration Welds	N/A <sup>3)</sup>
80-06	Engineered Safety Feature Reset Controls	7.2, 7.3
80-05	Vacuum conditions Resulting in Damage to Chemical Volume Control System Holdup Tanks	11.2
80-04	Analysis of a PWR Main Steamline Break with Continued Feedwater Addition	6.2
80-03	Loss of Charcoal from Standard Type II, 2-inch, Tray Absorber Cells	N/A <sup>2)</sup>

주 : 적용하지 않는 항목(N/A)에 대한 예외기준

- 1) 특정 원자로형이나 특정 공급사에 해당되는 항목
- 2) 신고리 3,4호기에 해당되지 않는 구조물, 계통 및 기기들의 항목
- 3) 발전소 운영 및 특정 발전소 설계에 해당되는 항목
- 4) 설계요건에 포함되지 않는 항목

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신고리 3,4호기 최종안전성분석보고서

표 1.8-4 (4 중 1)

미국 원자력규제위원회 표준심사지침(SRP)과의 차이점

SRP 절/제목		비고 또는 차이점 기술	절
2.5.2	Vibratory Ground Motion - Rev.2	신고리 3,4호기는 10 CFR 50, 부록 S를 따른다.	표 2.0-1, 2.5.2.6
3.6.2	Determination of Rupture Locations and Dynamic Effects Associated With the Postulated Rupture of Piping - Rev.2, July 1981	원자로건물 관통부 이외의 지역에 설치된 1등급 배관에 대해 응력범위 즉, 운전등급 A 또는 B에서의 하중 조합식에 대해 식(10)과 식(12) 또는 식(13)으로 계산된 1차 및 2차 응력을 합한 응력값이 2.4 Sm보다 큰 지점에서 중간과단을 가정한다.	3.6.2.1.4.1
3.7.3	Seismic Subsystem Analysis - Rev. 2	대체분석방법은 배관계통에 사용된다.	3.7.3.1
3.7.3	Seismic Subsystem Analysis - Rev. 2	지저구조물의 현저한 진동에 관해 개기 및 가기의 기본진동수의 뚜렷한 한도는 설정되지 않는다.	3.7.3.4, 3.7.3.8
3.10	Seismic and Dynamic Qualification of Mechanical and Electrical Equipment	품질검사는 특정기기 조달시 이루어진다. 검사방법 및 기준은 기술되어 있다.	3.10
4.2	Fuel System Design - Rev. 2, July 1981	<p>핵연료집합체 측방편차를 고려해서 핵연료집합체구조물에 제한요소를 적용해 측방핵연료봉의 특정제한은 제공되지 않는다.</p> <p>15장 사고해석은 핵연료파손에 대해 핵비등이탈회선기준을 사용한다.; 4.4 절에 기술된 95/95 특정허용핵연료 설계제한치를 사용하지 않는다.</p> <p>후조사프로그램은 부지특성 안전성 분석보고서에 기술될 것이다.</p>	4.2.1.2



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신고리 3,4호기 최종안전성분석보고서

표 1.8-4 (4 중 2)

SRP 절/제목		비고 또는 차이점 기술	절
4.4	Thermal and Hydraulic Design - Rev. 1, July 1981	핵연료고밀화의 영향은 무시될 수 있기 때문에 총열속인자 및 선출력 생성률 계산에 포함되지 않는다.	4.4.2.2
4.5.1	Control Rod Drive Structural Materials - Rev. 2, July 1981	90 Ksi 이상의 항복강도에서 제어봉 구동구조물재료는 제어봉구동장치 상부 배기밸브내의 강철볼, 전동기 집합체에 장치된 베어링인서트에만 사용이 제한된다.	4.5.1.2
4.6	Functional Design of Control Drive System - Rev. 1, July 1981	비필수요소들이 두 계통 사이에 인터페이스를 수반하지 않기 때문에 제어봉구동장치와 제어봉집합체의 격리는 요구되지 않는다.	4.6.2.2
5.2.1.1	Compliance with the Codes and Standards Rule, 10 CFR 50.55a - Rev. 2, July 1981	설계적용코드 및 발행판은 표 1.8-5에 기술된다.	1.8
5.2.3	Reactor Coolant Pressure Boundary Materials - Rev. 2, July 1981	일렉트로슬로그 용접은 원자로냉각재압력경계 기기들의 제조에 사용되지 않는다.	5.2.3.3



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신고리 3,4호기 최종안전성분석보고서

표 1.8-4 (4 중 3)

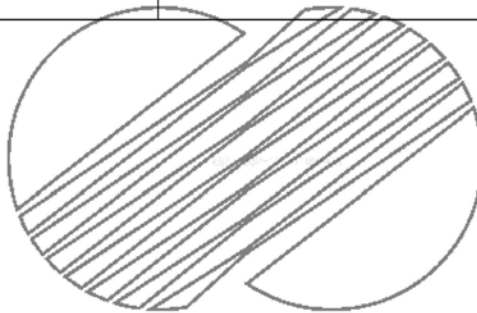
SRP 절/제목		비고 또는 차이점 기술	절
5.3.1	Reactor Vessel Materials - Rev. 2, April 1996	실제 원자로용기 재료는 재료구입시 시험될 것이다. 시험요건은 5.3.1.5절 에 기술된다.	5.3.1.5
5.4.1.1	Pump Flywheel Integrity (PWR)- Rev.1. July 1981	플라이휠 건전성을 입증하기위한 대안으로 선행호기 운전경험으로 검증된 대체 방법이 적용된다. 원자로냉각재펌프 플라이휠 설계는 5.4.1.1절에 기술된다.	5.4.1.1
6.2.2	Containment Heat Removal Systems - Rev. 4, October 1985	원자로건물내재장전수탱크 적용으로 원자로건물살수계통의 재순환운전모드 전환은 해당되지 않는다.	6.2.2.2, 6.5
6.2.4	Containment Isolation System - Rev. 2, July 1981	15장의 선량평가결과는 환기밸브가 닫힘 시간 30초를 만족함을 보여준다.	
6.4	Control Room Habitability System - Rev. 2, July 1981	운전원 재면실 및 주방은 비상구역 내부의 운전원 편의시설에 포함된다.	6.4
6.6	Inservice Inspection of Class 2 and 3 Components - Rev. 2, July 1981	가동중검사 프로그램은 6.6절에 요약되지만 검사될 기기들의 리스트는 사업주/운전원의 상세한 검사프로그램의 일부로서 제공될 것이다.	6.6
		용접구역에의 접근에는 특정한 제한이 가해진다.	6.6.2
11.1	Source Terms - Rev. 2, July 1981	방사성폐기물관리계통을 고려한 손익분석은 부지특성적용에 따른다.	11.1
		외부환경으로의 유출물 방출 계산에 사용된 방사성폐기물증대의 손익분석은 부지특성적용에 따른다.	
11.2	Liquid Waste Management Systems - Rev. 2, July 1981	액체방사성폐기물관리계통을 고려한 손익분석은 부지의 인구특성분석 때문에 부지특성적용에 따른다.	11.2.6.4

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신고리 3,4호기 최종안전성분석보고서

표 1.8-4 (4 중 4)

SRP 절/제목		비고 또는 차이점 기술	절
11.3	Gaseous Waste Management Systems - Rev. 2, July 1981	기체방사성폐기물관리계통을 고려한 손익분석은 부지특성적용에 따른다.	11.3.6.5
15.4.6	Chemical and Volume Control System Malfunction that Results in a Decrease in a Boron Concentration in the Reactor Coolant(PWR) - Rev. 1, July 1981	단일 작동기기파손 또는 운전자실수는 사고결과 영향에 무시될 수 있다.	15.4.6.1
16.0	Technical Specification - Rev. 1, July 1981	신고리 3,4호기 운영기술지침서의 내용은 표준운영기술지침서를 근거로 한다.	운영기술지침서



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신고리 3,4호기 최종안전성분석보고서

표 1.8-5 (4 중 1)

신고리 3,4호기에 적용되는 주요 규격 및 표준

규 격	출판연도	제 목
American Concrete Institute(ACI)		
318 349	1999 1997 and 2001*	Building Code Requirements for Structural Concrete Code Requirements for Nuclear Safety-Related Concrete Structures(*Appendix B only)
American Institute of Steel Construction(AISC)		
N690 (S327) S335	1994  1989	Specification for the Design, Fabrication, and Erection of Steel Safety-Related Structures for Nuclear Facilities Specification for Structural Steel Building - Allowable Stress Design and Plastic Design.
ANSI/American Nuclear Society(ANS)		
2.8 3.2 N13.1 51.1 55.4 56.2 57.1 57.2 58.2 58.8 58.9	1992 1994 1999 1983 1993 1984 1992 1983 1988 1994 1981	Determining Design Basis Flooding of Power Reactor Sites Administrative Control and Quality Assurance for the Operational Phase of Nuclear Power Plants Sampling and Monitoring Release of Airborne Radioactive Substances from the Stack and Ducts of Nuclear Facilities Nuclear Safety Criteria for the Design of Stationary PWR Plants Gaseous Radioactive Waste Processing Systems for Light Water Reactor Plants Containment Isolation Provisions for Fluid Systems after a LOCA Design Requirements for LWR Fuel Handling Systems Design Requirements for LWR Spent Fuel Storage Facilities at Nuclear Power Plants Design Basis for Protection of LWRs against Effects of Pipe Rupture Time Response Design Criteria for Safety-Related Operator Action Single Failure Criteria for LWR Safety Related Fluid Systems
ANSI/American Petroleum Institute(API)		
650	1993	Welded Steel Tanks for Oil Storage
ANSI/American Society of Civil Engineers		
7	1998	Minimum Design Loads for Building and Other Structures (ANSI A58.1)
American Society of Mechanical Engineers(ASME)		
BPVC	1995	Section II; Materials Specifications ; through 1997 Addenda



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신고리 3,4호기 최종안전성분석보고서

표 1.8-5 (4 중 2)

규 격	출판연도	제 목
BPVC	1995	Section III ; Rules for Construction of Nuclear Power Plant components ; through 1997 addenda
BPVC	1995	Section V ; Non-Destructive Examination ; through 1997 addenda
BPVC	1995	Section VIII ; Rules for Construction of Pressure Vessels ; through 1997 addenda
BPVC	1995	Section IX ; Qualification Standard for Welding and Brazing ; through 1997 addenda
BPVC	1995	Section XI ; Rules for Inservice Inspection of Nuclear Power Plant Components ; Editions and Addenda As Applicable ; through 1997 addenda
AG-1	1997	Code for Nuclear Air and Gas Treatment
B31.1	1998	Power Piping
OM	1995	Code for Operation & Maintenance of Nuclear Power Plants ; through 1997 Addenda
OM-S/G	2000	Standards and Guides for Operation and Maintenance of Nuclear Power Plants
NQA-1	1994	Quality Assurance Program Requirements for Nuclear Facility Applications ; through 1995 Addenda
ANSI/Institute of Electrical and Electronics Engineer(IEEE)		
7-4.3.2	1993	Standard Criteria for Digital Computer Systems of Nuclear Power Generation Stations
43	2000	Recommended Practice for Testing Insulation Resistance of Rotating Machinery
C.57.12.00	2000	IEEE Standard General Requirement for Liquid-Immersed Distribution, Power and Regulating Transformer
67	1990	Guide for Operation and Maintenance of Turbine Generators
100	2000	IEEE Standard dictionary of Electrical and Electronics Terms(Sixth Edition)
141	1993	Recommended Practice for Electrical Power Distribution for Industrial Plants (Correction Sheet-July 23 1997)
308	1991	Criteria for Class 1E Power Systems for NPGS
317	1983	Electrical Penetration Assemblies in Containment Structures for NPGS
323	1983	Qualifying Class 1E Equipment for NPGS
336	1985	Standard Installation, Inspection, and Testing Requirements for Power, Instrumentation and Contoil Equipment at Nuclear Facilities
338	1987	Standard Criteria for the Periodic Surveillance Testing of Nuclear Power Generation Station Safety Systems

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신고리 3,4호기 최종안전성분석보고서

표 1.8-5 (4 중 3)

규 격	출판연도	제 목
344	1987	Recommended Practices for Seismic Qualification of Class 1E Equipment for Nuclear Power Generation Stations
379	2000	Application of the Single Failure Criterion to NPGS Safety Systems
382	1996	Qualification of Actuators for Power Operated Valve assemblies with Safety-Related Functions of Nuclear Power Plants
383	1974	Standard for Type Test of Class 1E Electric Cables Field Splices, and Connection for Nuclear Power Generating Stations
384	1992	Criteria for Independence of Class 1E Equipment and Circuits
387	1995	Criteria for Diesel-Generator Units Applied as Standby Power Supplies for NPGSs
420	1982	Design and Qualification of Class 1E Control Boards, Panels, and Racks used in NPGSs
422	1986	Guide for Design and Installation of Cable Systems in Power Generation /Stations
497	1981	Criteria for Post-Accident Monitoring Instrumentation for NPGSs
572	1985	Standard for Qualification of Class 1E Connections Assemblies for Nuclear Power Generating Stations
603	1998	Criteria for Safety Systems for NPGSs
665	1995	IEEE Standard for Generation Station Grounding
741	1997	Criteria for Protection of Class 1E Power Systems and Equipment in NPGSs
828	1998	Standard for Software Configuration Management Plans
829	1998	Standard for Software Test Documentation
830	1998	Recommended Practice for Software Requirements Specifications
1008	1987	Standard for Software Unit Testing
1012	1998	Standard for Software Verification and Validation Plans
1028	1997	Standard for Software Reviews and Audits
1042	1987	Guide to Software Configuration Management
1074	1997	Standard for Developing Software Life Cycle Processes
ANSI/Instrument Society of America(ISA)		
S67.04	1994	Setpoints for Nuclear Safety-Related Instrumentation



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신고리 3,4호기 최종안전성분석보고서

표 1.8-5 (4 중 4)

규 격	출판연도	제 목
ANSI/National Fire Protection Association(NFPA)		
780	2000	Lightning Protection Code
101	1997	Safety to Life from Fire in Building and Structures
804	2001	Standard for Fire Protection for Advanced Light Water Reactor Electric Generation Plants
Electric Power Research Institute/NSAC		
108	1986	The Reliability of Emergency Diesel Generations at U.S. Nuclear Power Plants
Korea Electric Power Industry Code(KEPIC)		
KEPIC EC	2000	전선 및 전선용품 기술기준(Cable & Raceways)
KEPIC EE	2000	전기기기 기술기준(Electrical Equipment)
KEPIC EM	2000	계측 및 제어기기 기술기준(Measuring & Control Equipment)
KEPIC EN	2000	원자력전기 기술기준(Nuclear Electrical Equipment)
KEPIC ET	2000	송변배전 기술기준(Transmission, Transportation & Distribution)
KEPIC MB	2000	보일러 기술기준(Boilers)
KEPIC MD	2000	재료기술기준(Materials)
KEPIC ME	2000	비파괴시험 기술기준(Nondestructive Examination)
KEPIC MF	2000	원전기계기기 성능검증 기술기준(Qualification of Mechanical Equipment)
KEPIC MG	2000	일반기계 기술기준(General Mechanical Components)
KEPIC MH	2000	공조기기 기술기준(Nuclear Air & Gas Treatment)
KEPIC MI	2000	원전가동중검사 기술기준(Inservice Inspection of Nuclear Power Plant Components)
KEPIC MN	2000	원자력기계 기술기준(Nuclear Mechanical Components)
KEPIC MO	2000	가동중시험 기술기준(Inservice Testing of Nuclear Power Plant Components)
KEPIC MQ	2000	용접 기술기준(Welding and Brazing Qualification)
KEPIC MT	2000	터빈발전기 기술기준(Turbine Generators)
KEPIC QA	2000	품질보증 기술기준(Quality Assurance)
KEPIC SG	2000	일반구조 기술기준(General Structures)
KEPIC SN	2000	원자력구조 기술기준(Nuclear Structures)
KEPIC ST	2000	구조충척 기술기준(Extra Provisions for Structures)
KEPIC SW	2000	구조용접 기술기준(Structural Welding)



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신고리 3,4호기 최종안전성분석보고서

표 1.8-6 (4 중 1)

참조기준 적용기기 목록

계통명	기기명	주요 적용 기술기준
원자로냉각재 계통	파이롯트구동 안전방출밸브(POS RV)	ASME Sec. III Class 1, ASME QME-1 ASME B16.34, B16.5 API 527, IEEE 323, 344, 382, 946
	원자로냉각재펌프(RCP)	ASME Sec. III, IX, XI, ASME B16.5, B16.25, B16.11, B1.1, B31.1, Y14.1 ASME PTC 8.2, IEEE 344 ASTM A262, A800/A800M, D808, E112, E257, E350 IEEE/ASTM SI 10, NEMA MG 1, ANSI/HI
	원자로냉각재펌프 전동기(RCPM)	ASME B16.5, B16.11, B1.1, B31.1, Y14.1 IEEE 85, 112, 344, 383, 384, 620 IEEE/ASTM SI 10, NEMA MG 1
	원자로내부구조물 <sup>2)</sup>	ASME Sec. III, Class CS
	온도감지기 보호관 (Thermowell)	ASME Sec. III, Class 1
	증기발생기 노즐댐	ASME Sec. III, Class 1
원자로냉각재 계통 지지구조물	Snubber, Constant Effort Hanger, Rigid Sway Strut	ASME Sec. III, Class 1
안전감압배기계 통	분무기(Sparger)	ASME Sec. III Class 3
화학 및 체적제어계통	유량스위치(Flow Switch)	IEEE 344, ASTM A262
	필터(Filters)	ASME Sec. III Class 3, Sec. VIII, IX, XI ASME B16.11, B16.25, B36.19M ASTM A262, A36/A36M
	원심형충전펌프(CCP)	ASME Sec. III Class 3, Sec. XI, ASME QME-1 IEEE 112, 323, 334, 344, 946, 383, 384, 620 ANSI/HI, NEMA MG 1
	보조충전펌프(ACP)	ASME Sec. III Class 3, Sec. XI ASME PTC 7.1, ASME QME-1 IEEE 323, 344, 383, NEMA MG 1





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표 1.8-6 (4 중 2)

계통명	기기명	주요 적용 기술기준
급수계통	급수제어밸브(FWCV)	ASME B16.34, B31.1 NEMA 250, MG 1, WC-57, WC-70
	주급수격리밸브	ASME Sec. III Class 2
	급수 유량계 (Feedwater Flow Element)	ASME B16.11, B16.5, B31.1, ASME PTC-6 ASME Fluid Meters
주증기계통	주증기 격리밸브	ASME Sec. III Class 2
	주증기 안전밸브	ASME Sec. III Class 2
제어봉집합체 구동	제어봉구동장치 <sup>2)</sup>	ASME Sec. III Class 1, Sec. IV IEEE 323, 344
노내감시계통	가열접점열전대 탐침집합체	ASME Sec. III Class 1, Sec. II, Sec. IV IEEE 323, 344, ASTM E220
	가열접점열전대 MI 케이블	IEEE 323, 344, 383, 384 MIL-STD-202F
	노내계측기 집합체	ASME Sec. III Class 1, Sec. IV IEEE 344, 323, ASTM E142, E220, E230
	노내계측기 MI 케이블 및 케이블 트레이	IEEE 323, 344, 383, 384 AISC S327, MIL-STD-202F
핵증기공급제어 계통	공정제어 및 출력제어기기	IEEE 344, 383, 323
노심보호연산기 계통	노심보호연산기	IEEE 323, 344, 383, 384, 603, 7-4.3.2
원자로냉각재 펌프 속도감지기계통	회전속도감지기	IEEE 323, 344, 383
발전소보호계통	비교논리프로세서 등	IEEE 603, 344, IEC 61000-4-2
다양성보호계통	논리프로세서 등	IEEE 384
주요변수지시 및 경보계통-P	프로세서 등	IEEE 7-4.3.2, 323, 344, 384, 603
노외중성자속 감시계통	신호처리함 등	IEEE 323, 344, 384, 603
원자로건물 수소제어계통	피동측매형수소재결합기	IEEE 627
	수소점화기	IEEE 383, 344, 527, 627, NEMA WC-53
초음파 수위감시계통	Mid Loop 운전수위 계측기	IEEE 383



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신고리 3,4호기 최종안전성분석보고서

표 1.8-6 (4 중 3)

계통명	기기명		주요 적용 기술기준
액체방사성 폐기물계통	역삼투압 패키지	탱크(대기압)	API 620, API 650
		탱크(압력용기), 이온교환기, 여과기 등	ASME Sec. VIII
		열교환기	ASME Sec. VIII, TEMA
		배관 및 밸브	ASME B31.1
소내교류전력계 통	발전기 주차단기		ANSI C37.06, IEEE C37.013 IEEE C37.30
전기보온계통	전기보온설비 제어반 및 전기보온 케이블		IEEE 515, 622
인간기계연계 계통	기기제어기 등		IEEE 603, 7-4.3.2, 1028
기 타	안전방출밸브(SRV)		ASME Sec. III, XI ASME B16.5, B31.1, Y14.1, B16.34 ASME QME-1 ANSI/ASME N278.1, API-527 IEEE 344, ASTM A262, A800/A800M
	솔레노이드밸브(SOV)		ASME Sec. III Class 1, 2 ASME Sec. XI ASME B16.11, B16.25, B16.34, QME-1 NEMA 250, MSS SP-61, IEEE 344, 323, 382 ASTM A262, A800/A800M
	압력조절기밸브(PRV)		ASME B16.11, B16.34, Y14.1 ASTM A800/A800M
	공기구동제어밸브(POV)		ASME Sec. III, VIII, XI ASME B16.11, B16.34, B31.1, Y14.1 ASME QME-1 IEEE 344, 323, 382, 946, MSS SP-61 NEMA 250, MG 1 ASTM A262, A800/A800M



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신고리 3,4호기 최종안전성분석보고서

표 1.8-6 (4 중 4)

계통명	기기명	주요 적용 기술기준
기 타	온도감지기 보호관 (RTD Thermowell)	ASME Sec. III Class 1, 2, 3 ASME Y14.1, B16.11 IEEE 344, ASTM A262
	유량계(Flow Elements)	ASME Sec. III Class 2 IEEE 323, 344, 383, ASME B16.5 ASTM A262, A800/A800M
	제염장비	ASME B31.1, API 650
	단일스터드신장기 (Single Stud Tensioner)	NEMA 250, MG 1
	계기정비기기	IEEE 488.1
	차압식 스위치/지시계	ASME B16.5
	차압식 및 위치변위식 신호 전송기	IEEE 323, 344
	화학실험기기	IEEE 488.1
	특수케이블 B	NEMA WC55(ICEA S-82-552)
	전기시험장비	IEEE C57.12.90, 488.1
	MOV 시험 및 분석장비	IEEE 488.1
	전선관밀봉재집합체 (ECSA)	IEEE 317, 323, 344, 383, 572

1. 상기 기기는 해외구매품임.
2. 3호기는 WEC가 공급하며, 4호기는 두산중이 공급함.

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신고리 3,4호기 최종안전성분석보고서

표 1.8-7

신고리 3,4호기에 적용되는 ASME Sec. III 코드 케이스

번 호	제 목
N-4-11	Special Type 403 Modified Forgings or Bars, Class 1 and CS, 1981. 7. 13
N-60-5	Material for Core Support Structures, 1994. 2. 15
N-71-16	Additional Materials for Subsection NF, Classes 1, 2, 3 and MC Component Supports Fabricated by Welding, 1993. 2. 12
N-71-17	Additional Materials for Subsection NF, Classes 1, 2, 3 and MC Component Supports Fabricated by Welding, Section III, Division 1, 1995. 1. 1
N-122-2	Procedure for Evaluation of the Design of Rectangular Cross Section Attachments on Class 1 Piping, 1994. 4. 28
N-192-2	Use of Braided Flexible Connectors, Class 2 and 3, 1981. 9. 16
N-249-13	Additional Materials for Subsection NF, Classes 1, 2, 3 and MC Component Supports Fabricated without Welding, 1994. 5. 11
N-318-5	Procedure for Evaluation of the Design of Rectangular Cross Section Attachments on Class 2 or 3 Piping, 1994. 4. 28
N-391-1	Procedure for Evaluation of the Design of Hollow Circular Cross Section Welded Attachments on Class 1 Piping, 1983. 11. 28
N-392-3	Procedure for Evaluation of the Design of Hollow Circular Cross Section Welded Attachments on Class 2 and 3 Piping, 1994. 12. 12
N-411-1	Alternative Damping Values for Response Spectra Analysis for Class 1, 2 and 3 Piping, 1986. 2. 20
N-420	Linear Energy Absorbing Supports for Subsection NF, Classes 1, 2 and 3 Construction, 1985. 2. 14
N-474-2	Design Stress Intensities and Yield Strength Values for UNS N06690 with a minimum specific Yield Strength of 35 ksi, Class 1 Components, 1993. 12. 9
N-498-1	Alternative Rules for 10-Year System Hydrostatic Pressure Testing for Class 1 and 2 Systems, Section XI, Division I, 1994. 5. 11
N-596	Use of Alternate Reference Specimens Section III, Division 1, 1998. 3. 2
2142-2	F-Number Grouping for Ni-Cr-Fe Filler Metals Section IX (Applicable to all Sections, including Section III, Division 1, and Section XI), 2003. 8. 7
2143-1	F-Number Grouping for Ni-Cr-Fe, Classification UNS W86152 Welding Electrode Section IX(Applicable to all Sections, including Section III, Division 1, and Section XI), 1995. 6. 5



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표 1.8-8 (2 중 1)

미국 원자력규제위원회 정책현안(SECY-93-087) 적용성 분석

번 호	제 목	절
I.A	Use of a Physically Based Source Term	3.11, 6.5, 부록 15A
I.B	Anticipated Transients Without Scram	7.3, 7.7.1.1.11
I.C	Mid-Loop Operation	주 1)
I.D	Station Blackout	8.1.4.2, 8.4
I.E	Fire Protection	9.5.1
I.F	Intersystem Loss-of-Coolant Accident	부록 5E
I.G	Hydrogen Control	6.2.5
I.H	Core Debris Coolability	주 2)
I.I	High-Pressure Core Melt Ejection	주 2)
I.J	Containment Performance	주 2)
I.K	Dedicated Containment Vent Penetration	주 2)
I.L	Equipment Survivability	주 2)
I.M	Elimination of Operation-Basis Earthquake	2.5, 3.7
I.N	Inservice Testing of Pumps and Valves	3.9.6, 5.2.4, 6.6
II.A	Industry Codes and Standards	1.8
II.B	Electrical Distribution	8.2, 8.3
II.C	Seismic Hazard Curves and Design Parameters	주 3)
II.D	Leak-Before-Break	3.6.2.1.3, 3.6.3
II.E	Classification of Main Steamlines in Boiling Water Reactors	N/A
II.F	Tornado Design Basis	2.3.2.1
II.G	Containment Bypass	6.2.2, 부록 5F
II.H	Containment Leak Rate Testing	3.8.2.7
II.I	Post-Accident Sampling System	9.3.2
II.J	Level of Detail	N/A
II.K	Prototyping	N/A
II.L	ITAAC	N/A



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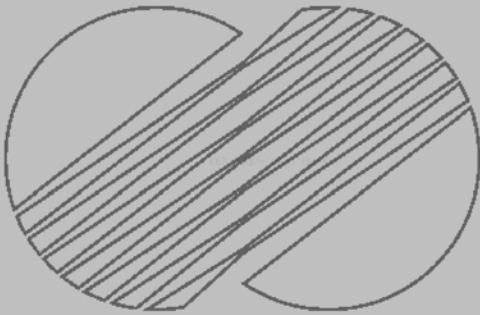
표 1.8-8 (2 중 2)

번 호	제 목	절
II.M	Reliability Assurance Program	N/A
II.N	Site-Specific Probabilistic Risk Assessments and Analysis of External Events	주 3)
II.O	Severe Accident Mitigation Design Alternatives	주 3)
II.P	Generic Rulemaking Related to Design Certification	N/A
II.Q	Defense Against Common-Mode Failures in Digital Instrumentation and Control Systems	7.2, 7.3, 7.7, 부록 7A
II.R	Steam Generator Tube Rupture	주 3)
II.S	PRA Beyond Design Certification	주 3)
II.T	Control Room Annunciator(Alarm) Reliability	7.7
III.A	Regulatory Treatment of Nonsafety Systems in Passive Designs	N/A
III.B	Definition of Passive Failure	N/A
III.C	SBWR Stability(Passive Design)	N/A
III.D	Safe Shutdown Requirements(Passive Design)	N/A
III.E	Control Room Habitability(Passive Design)	N/A
III.F	Radionuclide Attenuation(Passive Design)	N/A
III.G	Simplification of Offsite Emergency Planning	N/A
III.H	Role of the Passive Plant Control Room Operator	N/A

- 1) 별도보고서인 “정지위험도분석보고서”에 기술된다.
- 2) 별도보고서인 “중대사고분석보고서”에 기술된다.
- 3) 별도보고서인 “확률론적안전성평가보고서”에 기술된다.
- 4) N/A는 신고리 3,4호기 설계에 적용할 수 없는 항목을 나타낸다.



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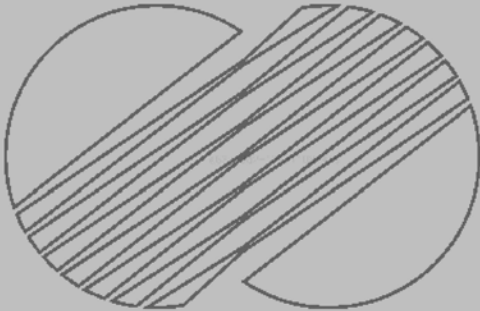
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신고리 3,4호기  
최종안전성분석보고서


부지 배치도

그림 1.2-1



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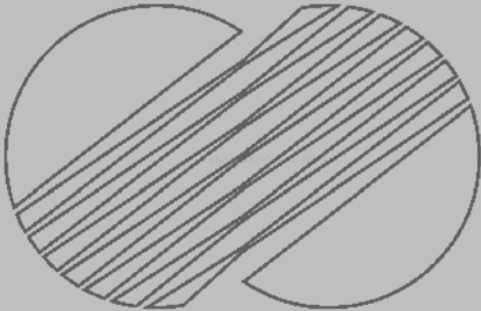



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	기기 배치도
	그림 1.2-1A



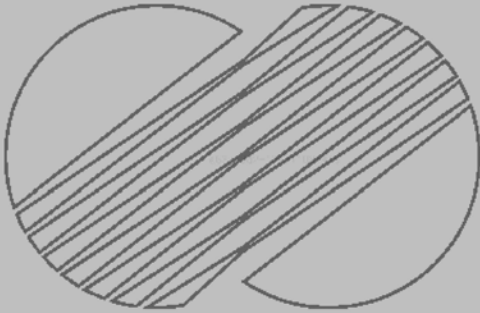


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부지 평면 계획	
그림 1.2-1B	

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한국수력원자력주식회사  
신고리 34호기  
최종안전성분석보고서

원자로건물 일반 배치도(A-A 단면도)

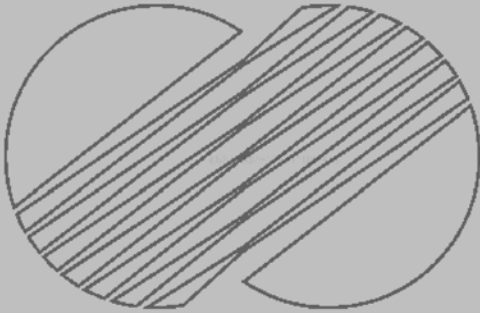
그림 1.2-2



한국수력원자력주식회사  
신고리 3,4호기  
최종안전성분석보고서

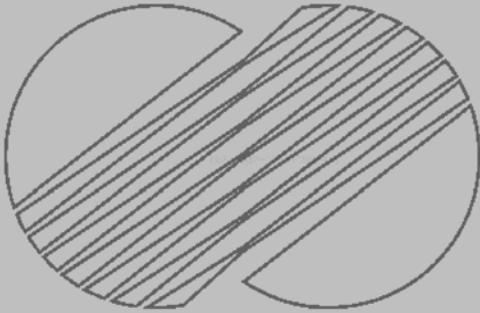
원자로건물 일반 배치도(B-B 단면도)


그림 1.2-3



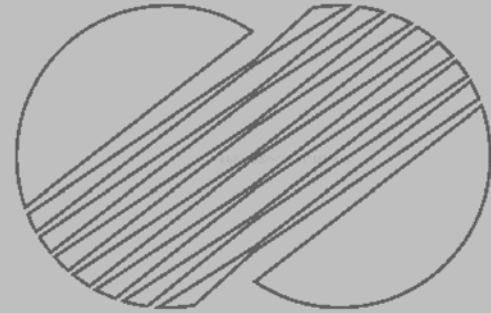
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정 1  
15. 10. 30



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원자로건물 일반 배치도 (EL. 69'-0" & 78'-0")	
그림 1.2-4	

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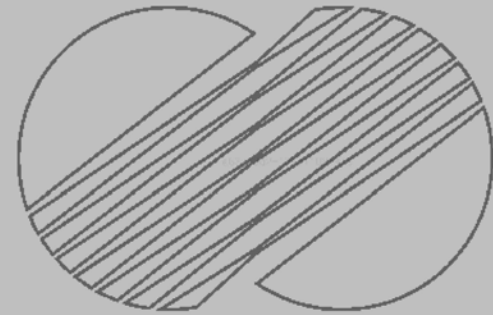


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신고리 3,4호기  
최종안전성분석보고서

원자로건물 일반 배치도 (EL. 100'-0")

그림 1.2-5

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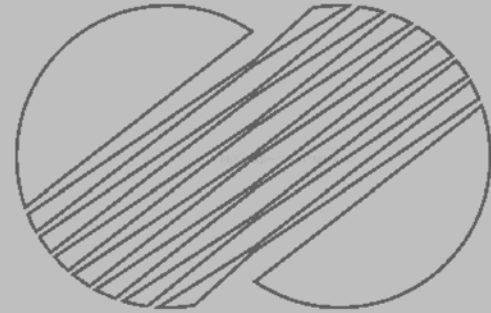


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신고리 3,4호기  
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원자로건물 일반 배치도 (EL. 114'-0")

그림 1.2-6

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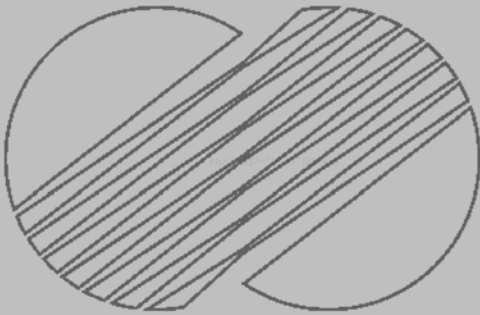
한국수력원자력주식회사  
신고리 3,4호기  
최종안전성분석보고서

원자로건물 일반 배치도 (EL. 136'~6")

그림 1.2-7

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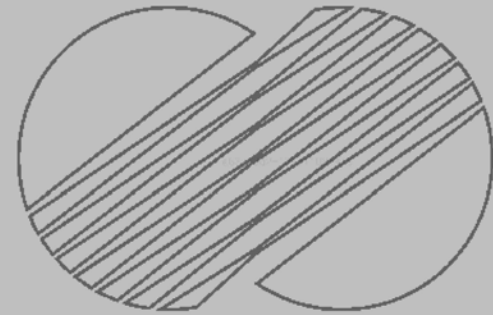
한국수력원자력주식회사  
신고리 34호기  
최종안전성분석보고서

원자로건물 일반 배치도 (EL. 156'-0")

그림 1.2-8



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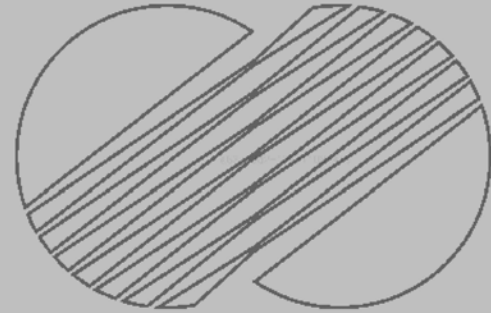
한국수력원자력주식회사  
신고리 3,4호기  
최종안전성분석보고서

보조건물 일반배치도(A-A 단면도)

그림 1.2-9

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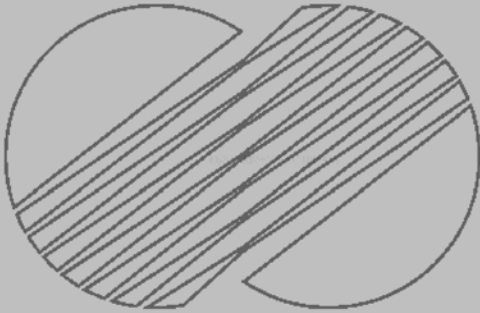
개정 1  
2015. 10. 30




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신고리 34호기  
최종안전성분석보고서

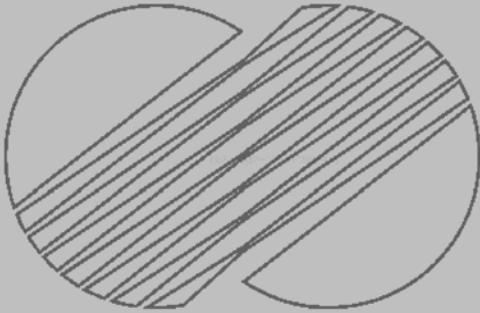
보조건물 일반배치도(B-B 단면도)

그림 1.2-10



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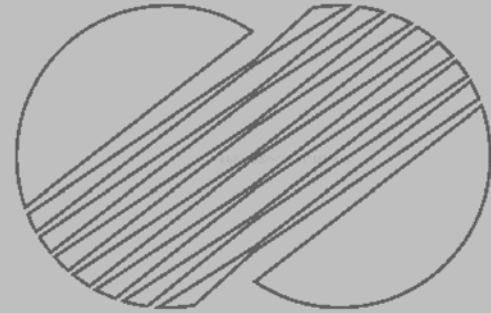


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신고리 3,4호기  
최종안전성분석보고서

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그림 1.2-12

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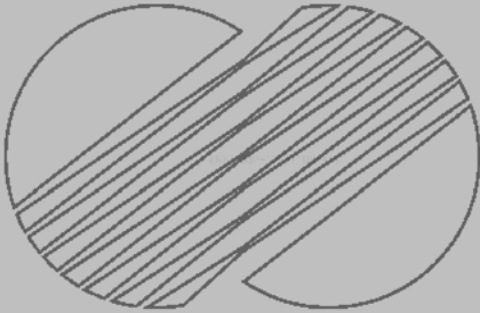
한국수력원자력주식회사  
신고리 3,4호기  
최종안전성분석보고서


보조건물 일반배치도(EL. 78'-0")

그림 1.2-13

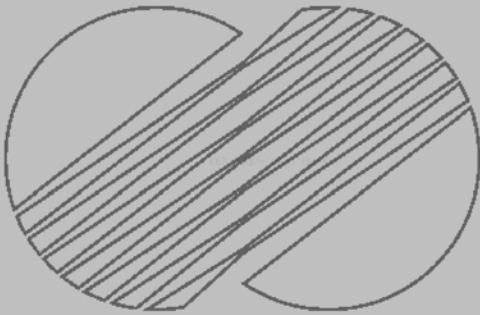
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5. 10. 30



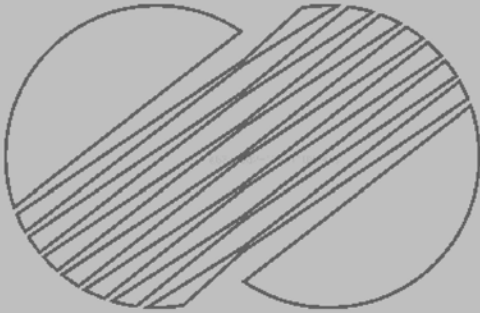
 한국수력원자력주식회사 신고리 3,4호기 최종안전성분석보고서	보조건물 일반배치도(EL. 100'-0")  그림 1.2-14
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	한국수력원자력주식회사 신고리 3,4호기 최종안전성분석보고서
보조건물 일반배치도(EL. 120'-0")	
그림 1.2-15	

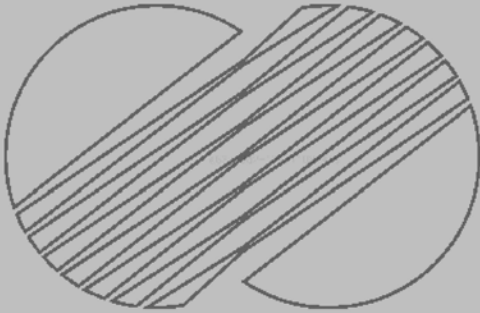





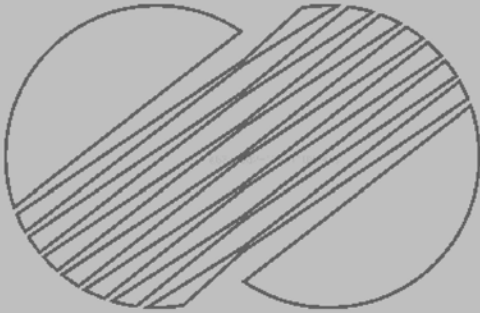
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보조건물 일반배치도(EL. 137'-6")	
그림 1.2-16	







 <div>한국수력원자력주식회사 신고리 3,4호기 최종안전성분석보고서</div>	<div>보조건물 일반배치도(EL. 156'-0")</div> <div>그림 1.2-17</div>
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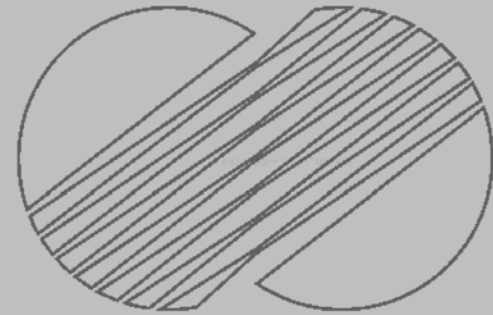


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최종안전성분석보고서

보조건물 일반배치도(EL. 172' -0")

그림 1.2-18

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보조건물 일반배치도(EL. 190'-0")

그림 1.2-19

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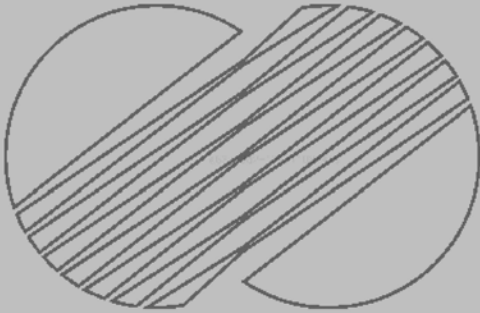
개정 1  
2015. 10. 30




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복합건물 일반 배치도(A-A 및 B-B 단면도)

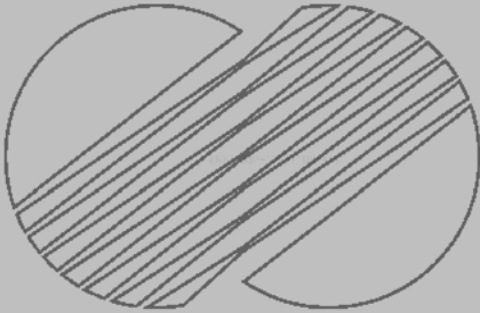
그림 1.2-20



	한국수력원자력주식회사 신고리 3,4호기 최종안전성분석보고서
복합건물 일반 배치도(EL. 63'-0")	
그림 1.2-21	

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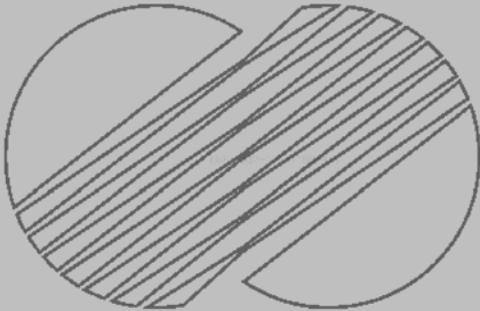



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복합건물 일반 배치도(EL. 77'-0")

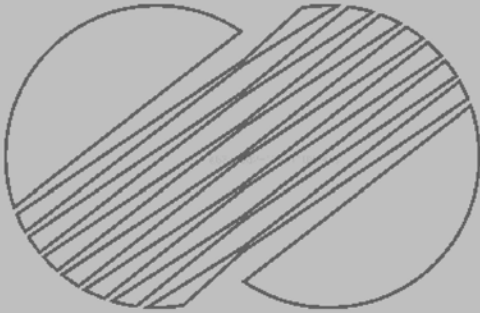
그림 1.2-22

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복합건물 일반 배치도(EL. 85'-0")	
그림 1.2-23	





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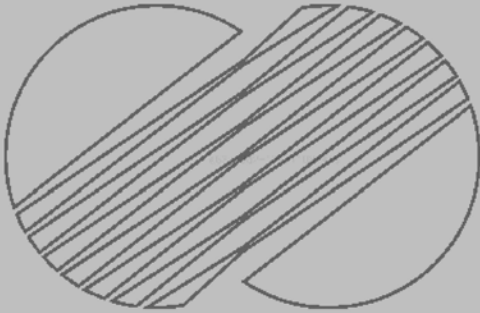
복합건물 일반 배치도(EL. 100'-0")

그림 1.2-24



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15. 10. 30



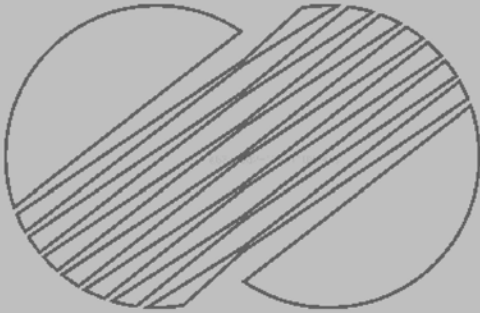
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복합건물 일반 배치도(EL. 120'-0")

그림 1.2-25

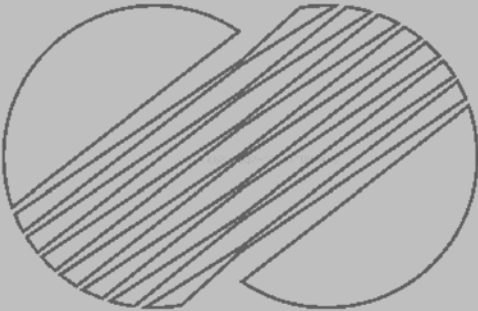
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복합건물 일반 배치도(EL. 139'-6")	
그림 1.2-26	

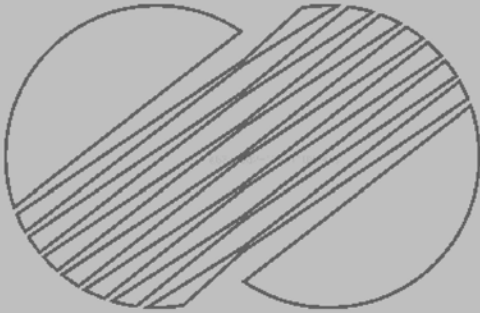
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복합건물 일반 배치도(EL. 156'-0")	
그림 1.2-27	



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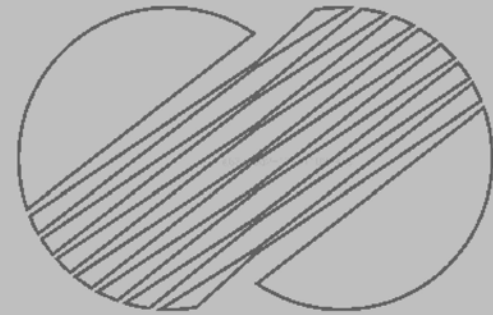



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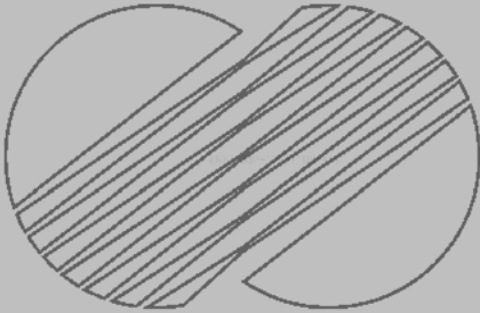
터빈건물 일반배치도(A-A 단면도)


그림 1.2-28

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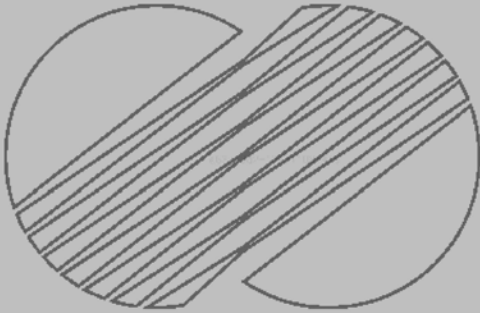


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	신고리 3,4호기
	최종안전성분석보고서
	터빈 건물 일반배치도(B-B 단면도)
	그림 1.2-29



	한국수력원자력주식회사 신고리 3,4호기 최종안전성분석보고서
터빈건물 일반배치도(EL. 73'-0")	
그림 1.2-30	

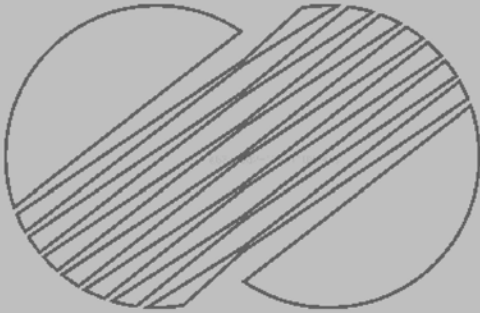




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터빈건물 일반배치도(EL. 100'-0")

그림 1.2-31



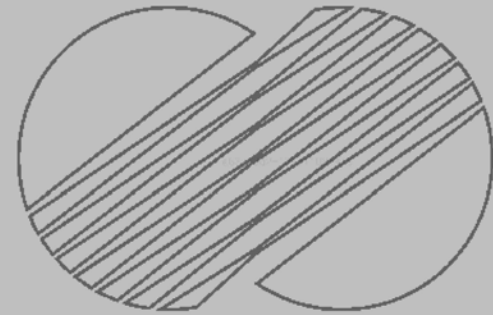
한국수력원자력주식회사  
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터빈건물 일반배치도(EL. 136'-6")

그림 1.2-32



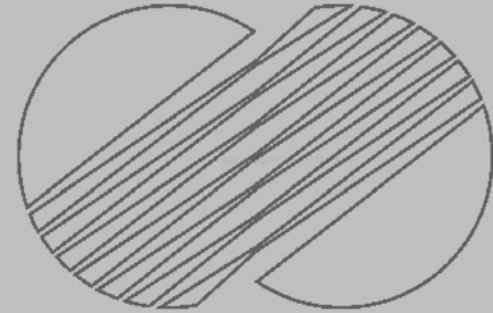
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터빈 건물 일반배치도(EL. 170'-0")

그림 1.2-33



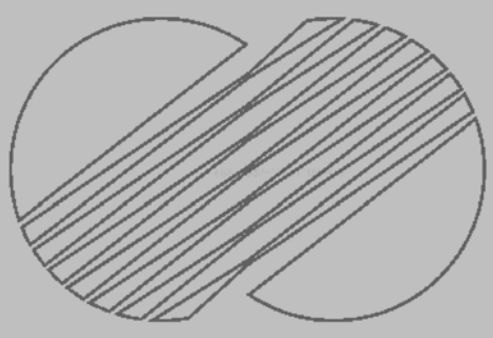
 <div>한국수력원자력주식회사 신고리 3.4호기 최종안전성분석보고서</div>	터빈 건물 일반배치도(EL. 204'-5")  그림 1.2-34
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배관 및 계장도 기호 및 약자

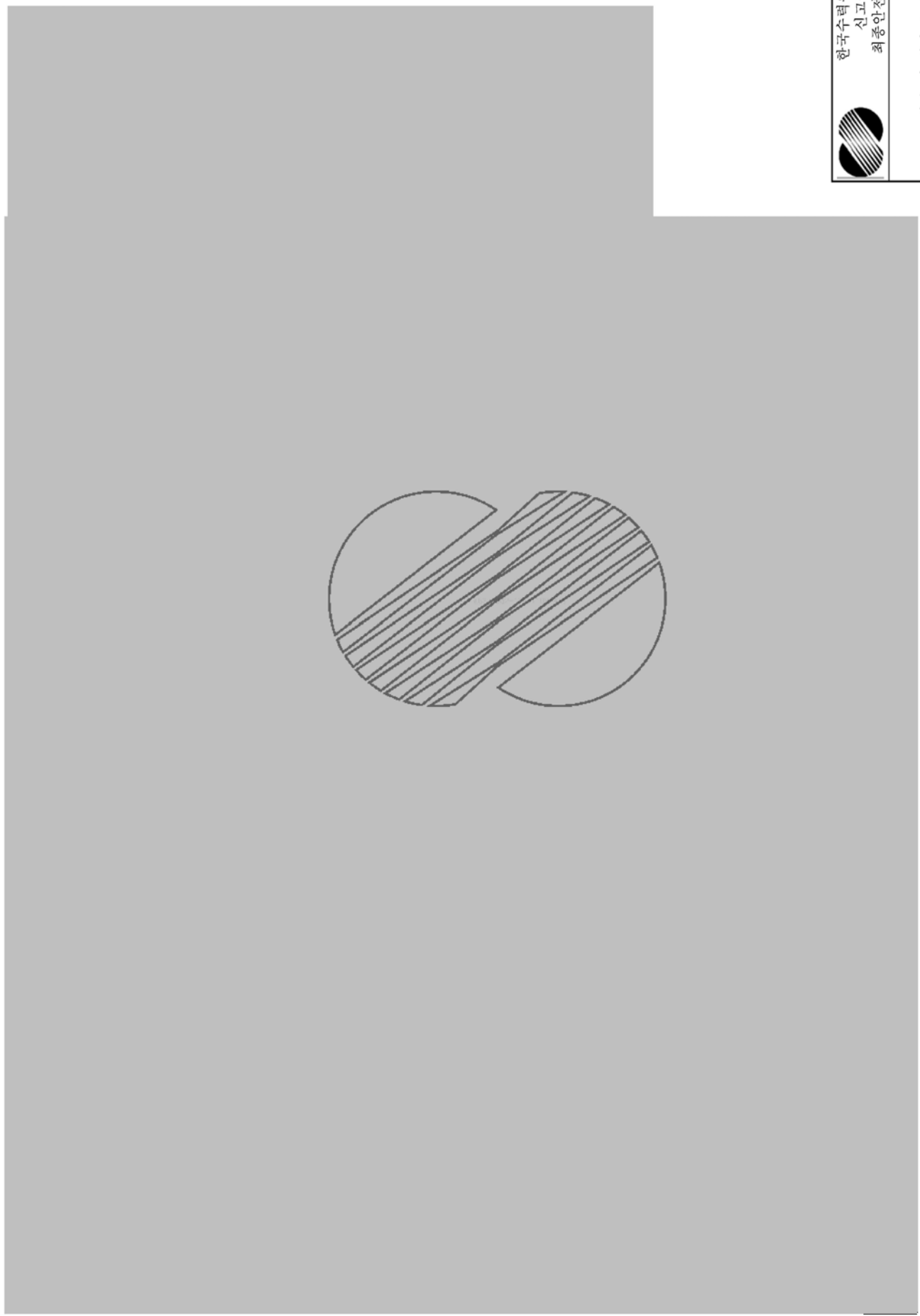
그림 1.7-1 (5 중 1)




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배관 및 계장도 기호 및 약자	
그림 1.7-1 (5 중 2)	

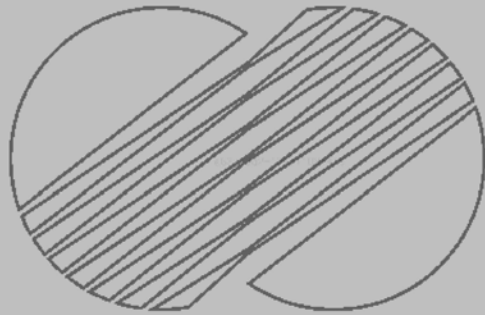




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최종안전성분석보고서

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그림 1.7-1 (5 중 3)



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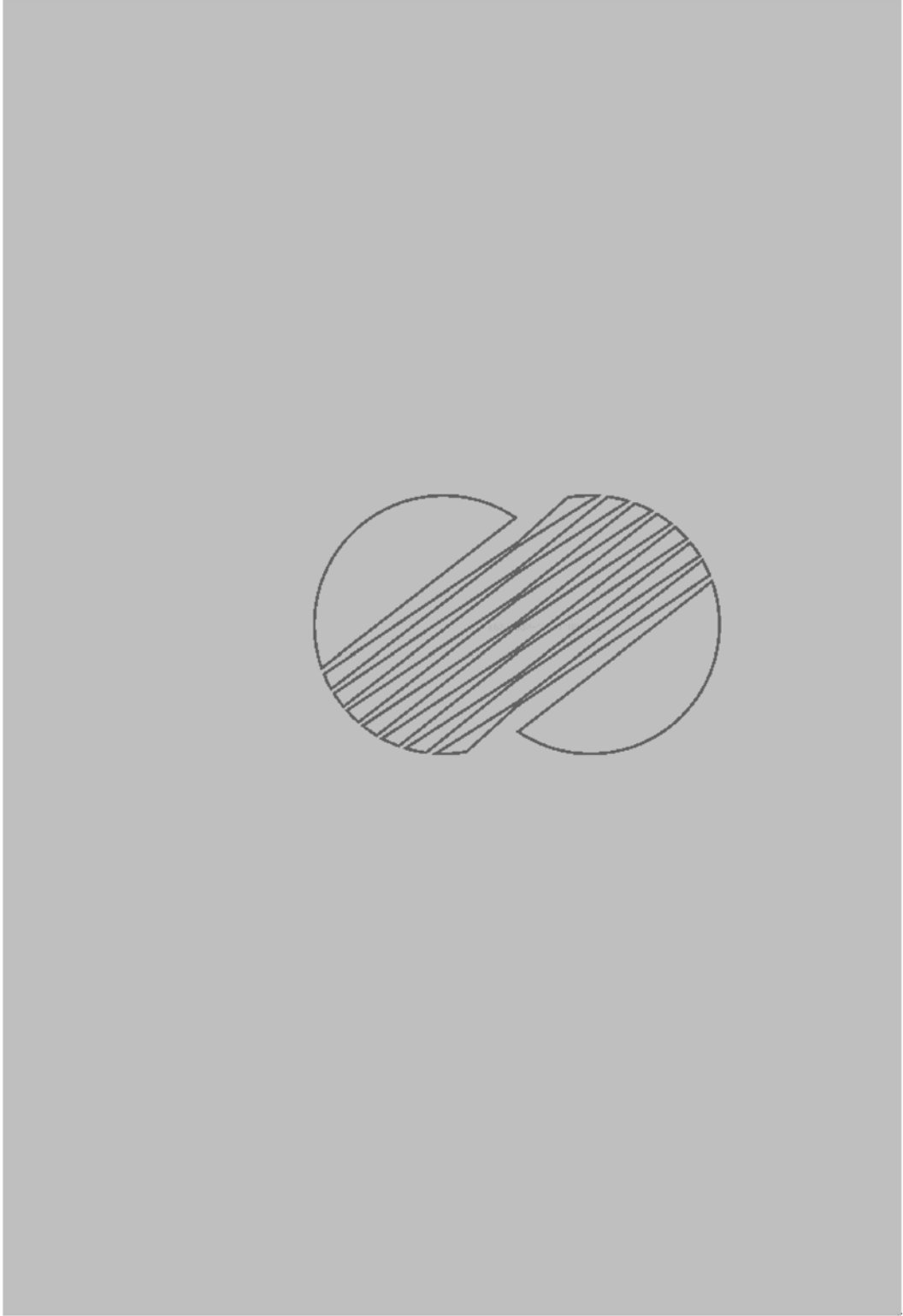




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최종안전성분석보고서

배관 및 계장도 기호 및 약자

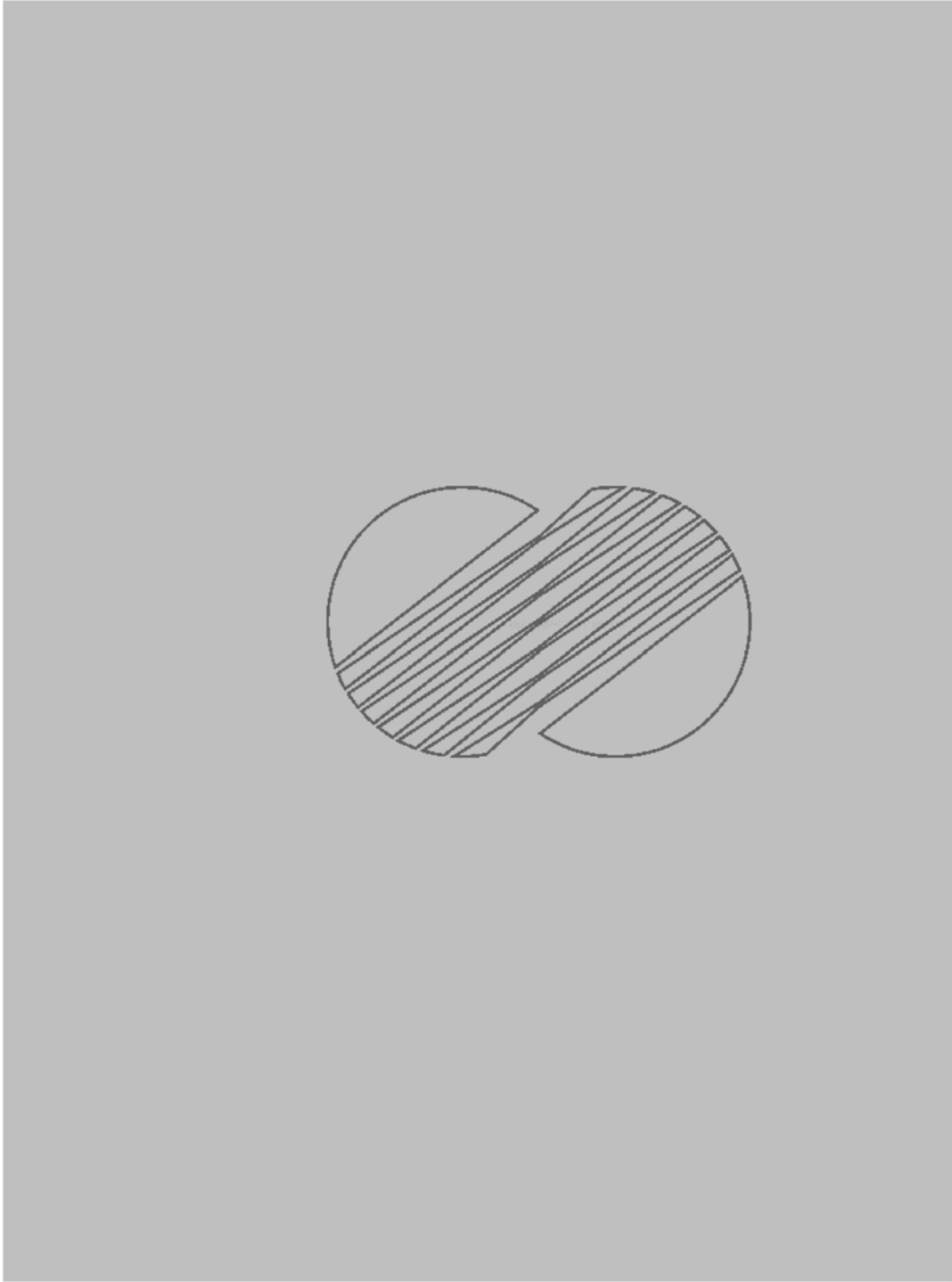
그림 1.7-1 (5 중 4)




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배관 및 계장도 기호 및 약자	
그림 1.7-1 (5 중 5)	